NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

# LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | DOCKET NUMBER (2) | PAGE (3) |
|---|---|---|
| Callaway Plant Unit 1 | 0 5 0 0 0 4 8 3 | 1 OF 0 2 |

TITLE (4)

Inadvertent Engineered Safety Feature Actuation

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| | | | | | | | | | | 0 5 0 0 0 |
| 0 9 | 3 0 | 8 4 | 8 4 | — 0 4 5 | — 0 0 | 1 0 | 2 9 | 8 4 | | 0 5 0 0 0 |

| OPERATING MODE (9) | 3 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | |
|---|---|---|---|---|---|
| POWER LEVEL (10) | 0 0 0 | 20.402(b) | 20.406(c) | X 50.73(a)(2)(iv) | 73.71(b) |
| | | 20.405(a)(1)(i) | 50.36(c)(1) | 50.73(a)(2)(v) | 73.71(c) |
| | | 20.405(a)(1)(ii) | 50.36(c)(2) | 50.73(a)(2)(vii) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| | | 20.405(a)(1)(iii) | 50.73(a)(2)(i) | 50.73(a)(2)(viii)(A) | |
| | | 20.405(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | |
| | | 20.405(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

| NAME | TELEPHONE NUMBER |
|---|---|
| Michael E. Taylor - Superintendent, Operations | AREA CODE 3 1 4 — 6 7 6 - 8 2 0 7 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | |
|---|---|---|---|---|---|---|---|---|---|---|---|
| | | | | | | | | | | | |
| | | | | | | | | | | | |

| SUPPLEMENTAL REPORT EXPECTED (14) | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|---|---|---|---|---|
| YES (If yes, complete EXPECTED SUBMISSION DATE) | X NO | | | | |


ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9/30/84 while in Mode 3, a Feedwater Isolation Signal (FWIS) was received as a result of a Reactor Trip signal and a low Reactor Coolant System (RCS) average temperature. The Feedwater Isolation Actuation performed as required upon initiation.

The Reactor Trip signal was received during the retest of a Reactor Trip bypass breaker. Due to incomplete initial conditions specified by the retest procedure, the Reactor Trip switch on the Main Control Board was in the tripped position when the bypass breaker was locally closed. The bypass breaker immediately reopened and initiated the Reactor Trip signal. This trip signal coupled with a RCS average temperature $\leq 564°F$ completed the logic for the FWIS.

Testing on the bypass breaker was suspended, the FWIS was reset, and the Feedwater Isolation valves were reopened. The retest was performed satisfactorily via a new retest procedure which corrected the deficiencies of the original procedure. Corrective action includes reviewing the incident with system engineers and incorporating the sequence of events into License Training. There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time was the public health or safety threatened.




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| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | PAGE (3) |
|---|---|---|---|---|---|
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | |
| Callaway Plant Unit 1 | 0 5 0 0 0 4 8 3 | 8 4 | — 0 4 5 | — 0 0 | 0 2 OF 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1029 CDT on 9/30/84, a Feedwater Isolation Signal (FWIS) was received as a result of a Reactor Trip signal and a low Reactor Coolant System (RCS) average temperature. The plant was in Mode 3 with an average RCS temperature of 558°F and a RCS pressure of 2235 psig. The Feedwater Isolation Actuation performed as required upon initiation.

The Reactor Trip signal was received during the retest of Reactor Trip bypass breaker SB-3. The retest was being performed subsequent to the replacement of the undervoltage and shunt trip assemblies on the Reactor Trip switchgear. At 1029 CDT, the Reactor Trip bypass breaker was locally closed but immediately reopened, thus initiating a Reactor Trip signal. With the average RCS temperature below 564°F and the Reactor Trip signal present, the FWIS logic was satisfied and the isolation occurred.

The testing on the Reactor Trip bypass breaker was suspended and the breaker was racked out. The FWIS was reset and the Feedwater Isolation valves were reopened at 1032 on 9/30/84.

Investigation of the incident revealed that the Reactor Trip switch on the Main Control Board was in the tripped position, thus causing the bypass breaker to immediately reopen. The procedure had not identified the required switch line-up needed to perform the retest. Upon identification and review of the deficiencies of the retest procedure, a new procedure was issued and performed satisfactorily on 10/1/84.

To prevent recurrence, this incident is to be reviewed with system engineers with particular emphasis on their responsibility to review and/or specify retest requirements. The expected completion date for this corrective action is 11/30/84. Also, the sequence of events surrounding this incident is to be incorporated into the training of licensed operators. No further corrective action is deemed necessary.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

Previous occurrences: none

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
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October 29, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-957

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-045-00
INADVERTENT ENGINEERED SAFETY FEATURE ACTUATION

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning an inadvertent Engineered Safety Feature actuation.

S. E. Miltenberger
Manager, Callaway Plant

MET/WRR/JWK/drs
Enclosure

cc: Distribution attached

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cc distribution for ULNRC-957

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