

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Pilgrim Nuclear Power Station - Unit 1

DOCKET NUMBER (2)
0 5 0 0 0 2 9 3

PAGE (3)
1 OF 0 2

TITLE (4)
Unplanned Actuation of Engineered Safeguards Feature (Containment Spray)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
09	29	84	84	0115	001	09	29	84			
									DOCKET NUMBER(S) 0 5 0 0 0		

OPERATING MODE (9) N

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(e)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(e)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii) (A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii) (B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: Richard M. Schifone - Plant Engineer

TELEPHONE NUMBER: 617 746-7910

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
A	B0	Z,Z,Z,Z	Z,Z,Z,Z	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9/29/84, during a refuel outage with the reactor vessel defueled, an unplanned actuation of the Containment Spray System occurred. As a result, approximately 10,000 gallons of water was sprayed into the drywell. The discharge was initiated when Maintenance personnel stroked the upper drywell outboard isolation valve during preventive maintenance.

The cause has been determined to be personnel error.

This event did not impact the health and safety of the public.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Pilgrim Nuclear Power Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 1 1 5	0 0	0 2	OF

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 9/29/84, during a refuel outage with the reactor vessel defueled, there was an unplanned actuation of the Containment Spray System. As a result, approximately 10,000 gallons of water was sprayed into the drywell.

Prior to the event, the "D" Residual Heat Removal (RHR) pump was running in the reactor cavity cleanup mode and electricians were performing preventive maintenance on the upper drywell spray header outboard isolation valve (MO-1001-23A). To check the motor operator limit switch position, electricians stroked the subject valve via remote, manual operation from the associated Motor Control Center (B-1756). They were not aware that the inboard valve (MO-1001-26A) was open and, when the outboard valve was also opened, the discharge from the "D" RHR pump was able to flow to the drywell.

The immediate corrective action was to secure the "D" RHR pump, manually close MO-1001-23A, and pump the drywell floor sumps to radwaste. The cause has been determined to be non-licensed personnel error in that the utility personnel performing the maintenance did not establish adequate communication with the Control Room prior to functionally testing the outboard valve. Therefore, the electricians were not aware of the actual plant conditions. Corrective action was to counsel appropriate personnel.

To reduce the possibility of a similar occurrence and to ensure communication with the Control Room, the valve maintenance procedure (3.M.4-10, Att. "C") has been changed. It now requires the Watch Engineer's signature on the valve actuator checklist prior to performing the test.

This event is considered an isolated occurrence which did not impact the health and safety of the public.

A search of records indicates no previous events of a similar nature at Pilgrim.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

October 29, 1984

BECo Ltr. #84-184

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Washington, D.C. 20555

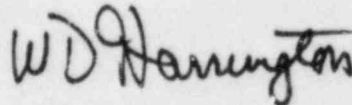
Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 84-015-00, "Unplanned Actuation of Engineered Safeguards Feature (Containment Spray)," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

RS:caw

Enclosure: LER 84-015-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue,
King of Prussia, PA 19406

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