9-83)	396					LIC	ENSE	E EVE	NT RE	PORT	(LER)		PROVE	REGULATO ED OMB NO 8/31/85		
FACILITY	NAME (1)										DOCKET NUMBER			_	E (3)
P	Hari	m N	luci	ear	Power St	ation (PNPS:) - U	nit 1			0 15 10 10	1012	1913	1 OF	012
TITL # 145		-	-							chnic		fications				
EVENT DATE (8) LER NUMBER (6)				AE	REPORT DATE (7) OTH				ER FACILITIES INVOLVED (6)							
MONTH DAY		YEAR	Y	YEAR SEQUENTIAL NUMBER		REVISION	MONTH DAY		YEAR	FACILITY NAMES			DOCKET NUMBER(S)			
													0 15	1010	101	
0 3	2 8	8 4	8	14	0 0 4	-011	110	110	8 4				0 5	51010	101	
028	PATING		TH	IE REPO	RT IS BUSMITTED	PURBUANT 1		EQUIREM	ENTE OF 10	CFR 9: /6	Check one or more	of the following) (1	1)			
POWEI LEVEI (10)		01	0	20.406 20.406	(6) Ha)(7)(8) Ha)(7)(8) Ha)(7)(W) Ha)(7)(W)	×	50.38(a 50.38(a 50.73(a 50.73(a 50.73(a	(1) (2) (2)(i) (2)(ii)			80.73(a)(2)(iv) 80.73(a)(2)(v) 80.73(a)(2)(vii) 80.73(a)(2)(viii)(30.73(a)(2)(viii)(30.73(a)(2)(x)		日	73.71(e) 73.71(e) OTHER (Spi below end in 366.A)	eify in Abi Text, NRC	strect C Form
							ICENSEE	CONTAC	FOR THIS	LER (12)						
NAME	R.	Sch	ifor	ne -	Plant En	gineer						6 1 1 17		16 1-	1719	1010
			-		COMPLETE	ONE LINE FOR	EACH C	OMPONEN	T FAILURE	DESCRIBE	D IN THIS REPO	RT (13)				
CAUSE	SYSTEM	co	MPONE	INT	MANUFAC- TURER	REPORTABLE TO NPROS				SYSTEM	COMPONENT	MANUFAC- TURER		NPROS		
D	SIB		1 ^R	1 1 0	121413	Y					111	111				
	1		1	1	111						111	1111				
					SUPPLEME	NTAL REPORT	EXPECT	ED (14)				EXPECT	ED	MONTH	DAY	YEAR
					Jamission DATE		7	NO NO				SUBMISSI DATE II	ION			

On 3/28/84. during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the Main Steam Safety Valves exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi \pm 13 psi. When tested, one valve lifted at 1209 psi, and the other lifted at 1155 psi.

The cause of this deviation has been determined to be the set pressure calibration method. The procedure allowed for the use of the nitrogen as a substitute test gas, in lieu of steam.

As a result of this determination, future safety valve testing/calibration will be performed using steam as the test medium.

8411090154 841010 PDR ADDCK 05000293 S PDR

IE22

NRC Form 356A (9-83)	LICENSEE EVENT RE	PORT (LER) TEXT CONTINU	ATION		GULATORY COMMISSION OMB NO. 3150-0104 31/85
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)	PAGE (3)
Pilgrim Nucle	ar Power Station -		YEAR	SEQUENTIAL REVISION	
Unit I	,	0 5 0 0 0 2 9 3	8 4	-01014 - 011	0 2 0 0 12

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 3/28/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the PNPS Dresser Main Steam Safety Valves, Model 3777, exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi ± 13 psi. The subject valves were being tested in accordance with the requirements of T.S. 4.6.D.1.

Both valves were tested twice. Valve #203-4A, Serial #BK6262 lifted at 1213 psi during Test 1 and 1209 psi during Test 2.

Valve #203-4B, Serial #BK6309 lifted at 1165 psi during Test 1 and 1155 psi during Test 2.

Subsequent evaluation has indicated that the procedure used for set pressure calibration was inadequate. The procedure allowed for the use of nitrogen as a test gas, in lieu of steam. It has been concluded that this substitution resulted in the safety valves' pressure setpoints being in error.

On 4/7/84, at Wyle Lab., the above-mentioned valves were calibrated for set pressure and tested for leakage with steam as the test medium. Both valves were certified as acceptable.

As a result of these findings, future safety valve testing/calibration will be performed with steam as the test medium as indicated by Station Procedure 3.M.4-7.

This event did not impact the health and safety of the public.

A search of records indicates no previous occurrences of a similar nature.

BOSTON EDISON COMPANY SOO BOYLSTON STREET BOSTON, MASSAGHUSETTS G2:99

WILLIAM D. HARRINGTON ----

October 10, 1984

2

QA X

CONTROL COPY

LER BY- UUY- 01 NEYWORDS: SAFETY

YES [

COMPONENT #

Q LIST #

VALUE SET POINT ESTOW

984202364 RMG CONTROL #

RECORD TYPE: A4.05

NON QA

BECo Ltr. #84-171

Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555

> Docket Number 50-293 License DPR-35

Dear Sir:

The attached update Licensee Event Report 84-004-01, "Safety Valve Setpoints Below Requirement of Technical Specifications," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

US Florungton

W. D. Harrington

Respectfully submitted.

PH: caw

Enclosure: LER 84-004-01

cc: Or. Thomas E. Murley Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

Standard BECo LER Distribution



AND DESCRIPTIONS	RIBUTION			
AME	WO.	W		
·u				
VIST				
Sheet		L		
		T		
		1		
		+		
	_	+-		