NRC FORM 366 U. S. NUCLEAR REGULATORY COMMISSION (7.77) LICENSEE EVENT REPORT Update Report - Previous Report Date 5/21/81 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: (1) 10 0 (2)1 (5) E II H 0 0 0 0 0 0 LICENSEE CODE CON'T REPORT 0 1 (6)0 15 (7)04 (8)1 0 2 5 8 4 0 10 i1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) 0 2 On 4/26/81, with Unit 1 shutdown and D it 2 operating at 99% power, 1B 0 3 diesel generator surveillance revealed that the standby service water system would not deliver rated flow. Plant service water could not be 0 4 supplied to 1B DG within 8 hours due to a valve malfunction; thus, 1B 0 5 DG was declared inoperable. Refueling operations were stopped. This is 0 6 not a repetitive occurrence. There were no effects on public health or 0 safety due to this event. SYSTEM CAUSE CAUSE COMP VALVE CODE COMPONENT CODE SUBCODE X (12) Z (13) IB (15 A P U M P X X 114 9 Z (16 SEQUENTIAL OCCURRENCE REVISION REPORT EVENT YEAR REPORT NO. CODE LER/RO TYPE NO. REPORT 8 0 3 5 0 3 1 X 1 NUMBER 32 ACTION EFFECT ON PLANT SHUTDOWN FUTURE ATTACHMENT SUBMITTED NPRD-4 FORM SUB PRIME COMP. COMPONENT (22 HOURS MANUFACTURER N b Y J 1 0 25 (24 A 15 (23 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The system's inability to deliver rated flow was due to the service water 1 0 isolation valve (1P41-F402A) not opening. The valve was repaired and 1 successfully retested and DG 1B was made operable on plant service water. The system was returned to an operable status prior to Unit 1 startup. See narrative for other corrective action. 1 4 METHOD OF FACILITY (30) OTHER STATUS % POWER DISCOVERY DESCRIPTION (32) (28) 0 0 H 0 NA B (31) Performed Surveillance Test 9 10 ACTIVITY CUNTENT 80 AMOUNT OF ACTIVITY (35 RELEASED OF RELEASE LOCATION OF RELEASE (36) Z (33) Z (34) 6 NA NA 10 11 PERSONNEL EXPOSURES 80 DESCRIPTION (39) NUMBER TYPE 37 Z 38 0 0 0 NA PERSONNEL INJURIES RO 8411080566 841025 PDR ADOCK 05000321 PDR ADOCK 05000321 DESCRIPTION (41) NUMBER (10) 0 0 0 NA 80 LOSS OF OR DAMAGE TO FACILITY IE22 (43) DESCRIPTION 9 Z (42) NA PUBLICITY NRC USE ONLY DESCRIPTION (45) **SSUED** N (44) NA 68 69 80 NAME OF PREPARER T. L. Elton, Acting Supt of Reg. Comp (912)367-7851

## NARRATIVE REPORT FOR LER 50-321/1983-035, Rev. 1

LICENSEE : GEORGIA POWER COMPANY FACILITY NAME : EDWIN I. HATCH DOCKET NUMBER : 50-321

### REQUIREMENT FOR REFORT:

This 30-day LER was formerly required by Tech. Specs. section 6.9.1.9.b due to the event showing that Unit 1 was not meeting the requirements of Tech. Specs. section 3.9.c and Unit 2 was not meeting the requirements of Tech. Specs. section 3.8.1.1.1.b.

## PLANT CONDITIONS AT THE TIME OF THE EVENT(S):

On 4/26/81, Unit 1 was in a refueling/torus modification outage and Unit 2 was in steady-state operation at approximately 2411 MWt (i.e., approximately 99% power).

#### DETAILED DESCRIPTION OF THE EVENT(S):

During performance of "DIESEL GENERATOR MANUAL START" procedure (HNP-2-3801) on 1B diesel generator, plant personnel determined that 'he standby service water system would not deliver rated flow. The 1B Diesel Generator was declared inoperable when plant service water could not be lined up to the diesel generator during performance of "PLANT SERVICE WATER PUMP AUTO FUNCTION OPERABILITY" procedure (HNP-1-3181).

## SUMMARY ASSESSMENT OF ACTUAL AND POTENTIAL SAFETY CONSEQUENCES AND IMPLICATIONS:

Unit 1 already had 1C Diesel Generator inoperable; thus, all refueling operations were stopped as required by Unit 1 Tech. Specs. section 3.9.C.2.

Unit 2 was placed into a 72 hour LCO. The requirements of Unit 2 Tech. Specs. 4.8.1.1.1.a and 4.8.1.1.2.a.4 were performed within one hour in order to comply with Tech. Specs. section 3.8.1.1., ACTION a. The health and safety of the public were not affected by this event.

# STATUS OF REDUNDANT OR BACKUP SUBSYSTEMS AND/OR SYSTEMS:

The 1A Diesel Generator and all independent A.C. electrical systems were operable at the time of this event on Unit 1. The remaining 2 Diesel Generators and all independent A. C. electrical systems on Unit 2 were operable throughout this event.

#### JUSTIFICATION FOR CONTINUED OPERATION:

N/A

IF REPETITIVE:

This is a non-repetitive event.

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### WHY CORRECTIVE ACTION DID NOT PREVENT RECURRENCE:

N/A

#### IMPACT TO OTHER SYSTEMS AND/OR UNIT:

This event had no impact on any other systems not listed in this report.

#### CAUSE(S) OF THE EVENTS(S):

The cause of the inability to line up pland service water to 1B Diesel Generator was due to the standby service water to Unit 1 division 1 service water isolation valve (1P41-F402A) not opening.

The investigation revealed the standby service water system did deliver rated flow to the 1B diesel generator. However, the instrumentation in use to measure flow did not indicate pump flow accurately at the discharge of the heat exchanger.

#### IMMEDIATE CORRECTIVE ACTION(S):

Investigation revealed stripped gears inside the 1P41-F402A actuator. The actuator on 1P41-F402A was repaired and the valve was returned to service.

## SUPPLEMENTAL CORRECTIVE ACTION(S):

Flow measurement instrumentation was installed just down stream of the standby plant service vacer pump prior to entering the 1B diesel generator heat exchanger and the orifice plate size was changed on the flow instrumentation on the 1B diesel generator heat exchanger exit. These modifications were made by Design Change Request 81-145. Applicable plant procedures were revised to make necessary changes to insure proper operation of the standby service water system.

ACTION(S) TO PREVENT RECURRENCE (IF DIFFERENT FROM CORRECTIVE ACTIONS):

N/A

Georgia Power Company Post Office Box 439 Baxley, Georgia 31513 Telephona 912 367-7781 912 537-9444

Edwin I. Hatch Nuclear Plant



October 25, 1984 GM-84-906

PLANT E. 1. HATCH Licensee Event Report Docket No. 50-321

United States Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1981-035, Rev. 1. This report was formerly required by by Hatch Unit 1 Technical ! Specification section 6.9.1.9.b.

upe C. Nix

General Manager

HCN/TLE/vlz

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