

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION I

IE Inspection Report No: 50-219/75-15 Docket No: 50-219

Licensee: Jersey Central Power & Light Company License No: DPR-16

Madison Avenue at Punch Bowl Road Priority: N/A

Morristown, New Jersey 07960 Category: C

Location: Forked River, New Jersey (Oyster Creek) Safeguards Group: N/A

Type of License: BWR 640 MW(e)

Type of Inspection: Special - Announced

Dates of Inspection: May 21-22, 1975

Dates of Previous Inspection: May 5-9, 1975

Reporting Inspector: E. P. Jernigan, Reactor Inspector Date: June 6, 1975
E. P. Jernigan, Reactor Inspector

Accompanying Inspectors: None Date: _____

_____ Date: _____

_____ Date: _____

_____ Date: _____

Other Accompanying Personnel: None Date: _____

Reviewed By: R. C. Haynes Date: June 6, 1975
R. C. Haynes, Senior Reactor Inspector

B/P/10

SUMMARY OF FINDINGS

Enforcement Action

A. Items of Noncompliance

None

B. Deviations

None

Licensee Action on Previously Identified Enforcement Matters

Not inspected.

Design Changes

Not inspected.

Unusual Occurrences

None reported.

Other Significant Findings

A. Current Findings

1. Acceptable Areas

a. Valve Wall Thickness Verification Program

The valve wall thickness verification program required by the RO:I letter of June 22, 1972 is approximately 52% complete. The documentation for that part of the program which has been completed was inspected and was found to be acceptable. The documentation reviewed by the inspector included the following:

1. Procedure NSD-V-3-010, Rev. 0, dated February 1, 1973.
2. Personnel Qualifications.
3. Data of Completed Valves.
4. Wall Thickness Data of 22 Safety Relief Valves.

(Details, Paragraph 2)

2. Unresolved Item

a. Scope of Valve Wall Thickness Verification Program

The scope of valve wall thickness verification program established by the licensee is based on a sample number of valves. The inspector informed the licensee that all of the valves designated in the RO:I letter of June 22, 1972 were to be inspected. This item is considered to be unresolved. (Details, Paragraph 3)

B. Status of Previous Unresolved Items

The unresolved item listed below has been resolved.

1. Ultrasonic and eddy current inspection data as originally reported for repairs to in-core housing weld No. 28-05.

(Ref. IE:I Inspection Report 50-219/74-12)

(Details, Paragraph 4)

Management Interview

A management interview was held at the site on May 23, 1975 with the following attendees:

Jersey Central Power & Light Company

- D. Ross, Manager, Generating Stations-Nuclear (telephone contact only)
- D. Reeves, Chief Engineer
- R. Swift, Maintenance Engineer
- J. Carroll, Plant Superintendent

Items discussed are summarized below:

A. Purpose of Inspection

The inspector stated that the purpose of this inspection was to audit the licensee's compliance with valve wall verification requirements outlined in the RO:I letter dated June 22, 1972.

In addition, this inspection included a review of test data and the licensee's evaluation of this data with respect to repairs performed on in-core housing 28-05 as previously reported in RO:I Inspection Report No. 50-219/74-12.

B. Valve Wall Thickness Verification Program and Scope of Program

The items discussed are as identified under Current Findings in the Summary of Findings of this report. (Details, Paragraphs 2 and 3⁴)

C. Review of Previous Unresolved Items

The item discussed concerned the review of certain nondestructive examination data and the licensee's evaluation thereof of tests associated with repairs to in-core housing 28-05. This item is considered to be resolved. (Details, Paragraph 4)

DETAILS

1. Persons Contacted

Jersey Central Power & Light Company

J. Carroll, Plant Superintendent
D. Reeves, Chief Engineer
R. Jacobstein, Project Engineer
R. Swift, Maintenance Engineer

2. Valve Wall Thickness Verification Program

a. Procedure Review

The inspector reviewed the licensee contractor's procedure, NSD-V-3-010, Rev. 0, dated February 1, 1973. The procedure, Ultrasonic Wall Thickness Measurement of Nuclear Piping System Valves, reflected those parameters necessary to obtain required results.

b. Personnel Qualifications

The inspector reviewed the contractor's personnel certificates of qualifications of all inspectors making valve wall thickness measurements and were in compliance with the requirements of referenced standard, SNT-TC-1A.

c. Data Review

The inspector audited the valve wall thickness measurement data of the 43 completed valves. All data had been evaluated and accepted by the licensee.

The data supplied and evaluation results appeared to be sufficient in content and detail.

d. Reactor Safety Valves

The inspector reviewed the mechanical wall thickness measurement data of the 22 safety valves, including 6 spare valves.

Valve design precludes the possibility of ultrasonically measuring the wall thickness of the pressure boundary portion of the valve. The pressure boundary of these valves are machined nozzles which are fabricated to closely controlled tolerances to permit assembly. After disassembly, nozzles of 5 valves were checked by the licensee for compliance to detail design requirements and were found to be within drawing tolerances. The remaining 17 valve nozzles were checked assembled and they were deemed to meet design requirements. The

inspector reviewed the nozzle measurement data and found the data met specification requirements.

3. Scope of Valve Wall Thickness Verification Program

The licensee's valve wall thickness verification program is comprised of a total of 73 valves, exclusive of the reactor safety valves. The scope of the licensee's program was to inspect a representative sample of each type of valve as stated in their letter to the Regional Office, dated July 21, 1972. The licensee was informed by the inspector that all of the valves designated in the RO:I letter, dated June 22, 1972 were to be inspected to meet the intent of this verification program.

This item is considered to be unresolved pending completion of the wall thickness verification program for the remaining valves. The licensee's program for the verification of the wall thickness of the 22 reactor safety valves is considered to be complete and meets the intent of the RO:I letter.

4. In-Core Housing 28-05 Repairs

The inspector reviewed the licensee's letter of September 10, 1974, which clarified the previous ultrasonic report for in-core housing weld No. 28-05. In addition, a pressure test of this weld was performed during the May 1975 outage and no leakage was observed.

The inspector also reviewed the licensee's inspection contractor's formal evaluation report dated July 30, 1974 with respect to certain indications found during eddy current tests of the housing. The report stated that during eddy current testing of the housing tube material, slight pitting was detected in the tube's I.D. which was less than 5 percent of the tube wall thickness. The pitting was within acceptable limits according to the contractor's evaluation report to the licensee.

This item is considered to be resolved.