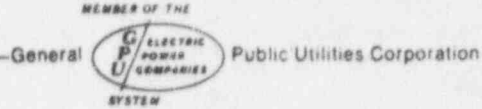


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# Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111



January 3, 1975

Mr. James P. O'Reilly, Director  
Directorate of Regulatory Operations, Region 1  
United States Atomic Energy Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly:

Subject: Oyster Creek Station  
Docket No. 50-219  
Response to DRO Bulletin No. 74-16

In DRO Bulletin No. 74-16 dated December 13, 1974, you alerted us to the possibility of having improperly machined pistons in Colt Industries (Fairbanks-Morse) diesel-generators. At our Oyster Creek Nuclear Generating Station, we presently utilize two different types of diesel engines. The emergency diesel-generators were manufactured by the Electro-Motive Division of General Motors Corporation, LaGrange, Illinois. The fire diesel engines were manufactured by Cummins Engine Company, Inc., Columbus, Indiana. Since these engines are not of the type described in your bulletin, no further action is planned on our part.

Very truly yours,

Donald A. Ross  
Manager, Generating Stations-Nuclear

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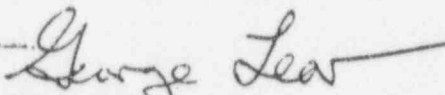
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- (c) The torus fatigue characteristics when subjected to forces resulting from opening of relief valves.
  - (d) The predicted maximum number of relief valve openings, singly and collectively, during plant life.
  - (e) Surveillance requirements, including frequency of inspections, for verification of torus structural integrity.
2. For the steam quenching vibration phenomenon, operating procedures should be developed, and changes to the Technical Specifications should be proposed to preclude the development of elevated temperatures of the torus pool water and provide for inspection of the torus as appropriate to identify any damage in the event of an extended relief valve operation. In this effort, consider the results of your review required by Regulatory Operations Bulletin 74-14, "BWR Relief Valve Discharge to Suppression Pool", dated November 11, 1974, and the interim recommendations of the General Electric Company to BWR owners (References 4 and 5).
3. Submit a description of your program and your proposed changes to the Technical Specifications. A suggested review schedule with major milestones and dates is enclosed for your consideration; if these dates are not satisfactory, adjustments may be possible after discussion with us. If you do not intend to initiate a program for verification of the torus and/or proposed surveillance requirements, we require that you submit appropriate justification.

Your submittals are requested in accordance with the proposed schedule. Three signed and notarized originals and thirty-seven (37) copies of your submittals are needed for our review.

Sincerely,



George Leat, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

Enclosure:  
Proposed Schedule of  
Major Activities

cc:  
See next page

THIS REQUEST FOR GENERIC INFORMATION WAS APPROVED BY GAO UNDER A BLANKET CLEARANCE NO. B-180225 (RO072); THIS CLEARANCE EXPIRES JULY 31, 1977.

*B/19/12*

cc: w/enclosure

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ENCLOSURE

SCHEDULE OF MAJOR ACTIVITIES

TORUS STRUCTURE INTEGRITY

<u>Activity</u>	<u>Date</u>
Letter to Licensees	February 15, 1975
Responses of Licensees	
w/Proposed Technical Specifications Revising Temperature Limits	March 31, 1975
w/Proposed Program for Maintaining Integrity of the Torus	May 30, 1975
Review of Responses and Issuance of Technical Specification Changes on Temperature Limits	August 29, 1975
Questions to Licensees Regarding Program for Maintaining Integrity of Torus	September 26, 1975
Licensee's Response with Proposed Technical Specifications on Surveillance Requirements	November 21, 1975
Review of Licensee's Response and Proposed Technical Specifications Surveillance Requirements	January 2, 1976
Technical Specification Change on Surveillance Requirements Issued	February 2, 1976
Expected Completion of Modifications	February 2, 1976