

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	PAGE (3) 1 OF 0 2
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TITLE (4)
Vacuum Breaker Surveillance Test Completed Late.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)
0 1	1 9	8 5	8 5	0 0 5	0 0	0 2	1 5	8 5				0 5 0 0 0
												0 5 0 0 0

OPERATING MODE (9) 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)						
	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)						
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.405(a)(1)(iii)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)							
	20.405(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)							
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME L.A. Kuczynski - Nuclear Plant Specialist, Level III		AREA CODE 7 1 7	5 4 2 - 3 7 5 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
*									

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single-space typewritten lines) (16)

On January 19, 1985, following a reactor scram due to turbine high vibration (see LER 85-003), which resulted in safety-relief valve (SRV) actuations, the required suppression chamber-drywell vacuum breaker surveillance test was completed after the time limit of Technical Specification Surveillance Requirement 4.6.4.b.1 had expired. The procedure which details Operations personnel actions following a reactor scram will be revised to include the requirement to perform the surveillance test within two hours of an SRV actuation. Additionally, the event will be reviewed with all licensed operators.

*Not applicable; no component failure

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On January 19, 1985, the Unit experienced a reactor scram following a turbine trip on high vibration (see LER 85-003). Safety-relief valves (SRV's) 'B', 'C', and 'E' lifted as a result. Technical Specification Surveillance Requirement 4.6.4.b.1 calls for the cycling of the suppression chamber-drywell vacuum breakers within two hours after any discharge of steam to the suppression chamber from the SRV's. Due to a cognitive error by Operations personnel (licensed, utility), the surveillance test which cycles the vacuum breakers was completed two hours and forty-five minutes after the Surveillance Requirement time limit had expired.

A revision will be made to the procedure which details Operations personnel actions following a reactor scram to include requirements that the vacuum breaker surveillance test be completed within two hours of an SRV actuation. Additionally, this event will be reviewed with all licensed operators.



Pennsylvania Power & Light Company

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February 15, 1985

U.S. Nuclear Regulatory Commission
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SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 85-005-00
ER 100450 FILE 841-23
PLAS-040

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 85-005-00. This event was determined reportable per 10CFR50.73(a)(2)(i), in that a vacuum breaker surveillance test was completed after the Technical Specification time limit had expired.

H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

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