

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) BYRON STATION	DOCKET NUMBER (2) 0 5 0 0 0 4 5 4	PAGE 3 1 OF 0 2
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TITLE (4)
FAILURE TO PERFORM SURVEILLANCE WITHIN TIME LIMIT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					
0	1	17	8	5	8	5	0	1	2	0	0	0	5	0

OPERATING MODE (8) **3**

POWER LEVEL (10) **0 0 0**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. Check one or more of the following: (11)

20.402(b)	20.408(a)	50.73(a)(2)(vi)	73.71(b)
20.408(a)(1)(i)	50.38(a)(1)	50.73(a)(2)(vi)	73.71(a)
20.408(a)(1)(ii)	50.38(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)
20.408(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(iii)	50.73(a)(2)(viii)(A)	
20.408(a)(1)(iv)	50.73(a)(2)(iv)	50.73(a)(2)(viii)(B)	
20.408(a)(1)(v)	50.73(a)(2)(iv)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Larry Alexander	TELEPHONE NUMBER extension 388
AREA CODE 8 1 5	2 3 4 5 4 4 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	W	I I S V	M 1 2 0	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 400 words - i.e. approximately fifteen single-space typewritten lines) (16)

The Tech Spec surveillance for Steam Generator Blowdown Containment Isolation Valves was found past its critical date with respect to valve LSD005C. The valve was declared inoperable on 1-17-85 at 1200, and the Action Requirement was initiated. The Tech Staff individual responsible for performing this surveillance delayed in it's execution to allow maintenance to complete work on other Steam Gen. Isolation Valves which are contained in the same surveillance. The surveillance program was changed to include the frequency on components requiring an accelerated testing interval and the Tech Staff Supervisor and his Assistants will be made aware of surveillances approaching their critical date.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 1-17-85, with the plant operating in Mode 3, the Tech Spec surveillance for the Steam Generator Blowdown Containment Isolation Valves was past its critical date with respect to valve 1SD005C. The valve was declared inoperable on 1-17-85 at 1200, which prohibited Rad/Chem from obtaining samples from the 1B Steam Generator.

The critical date was exceeded due to the improper valve being tested in the surveillance conducted on 1-8-85. The Tech Staff Engineer responsible for the system was notified of the error on 1-9-85, but he delayed reperforming the surveillance since Maintenance had work underway on other valves mentioned in the procedure.

Actually a partial surveillance could have been performed on valve 1SD005C without interfering with the maintenance on the other valves. The surveillance was reperformed on 1-17-85 missing the 1-16-85 critical date.

When the surveillance was reperformed, valve 1SD005C failed the acceptance criteria. Maintenance repaired the valve and the surveillance was performed with satisfactory results. The valve was inoperable for approximately 36 hours.

Plant and public safety were not affected as the valve was closed, which is the ESF position, during the time it was inoperable.

The surveillance program was modified to list components requiring an increased testing frequency to inform supervisory personnel of surveillances which are nearing their due date.

This is the first occurrence in which this surveillance exceeded its critical date.



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

February 11, 1985

LTR: BYRON 85-0218

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2)(i) which requires a 30 day written report.

This report is number 85-012-00, Docket No. 50-454.

Very truly yours,

R. E. Querio

R. E. Querio
Station Superintendent
Byron Nuclear Power Station

REQ/da

Enclosure: Licensee Event Report No. 85-012-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
CECO Distribution List

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