

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JUN 20 1975

DOCKET NO.: 50-219

LICENSEE: JERSEY CENTRAL POWER AND LIGHT COMPANY

FACILITY: OYSTER CREEK NUCLEAR GENERATING STATION

SUMMARY OF MEETING WITH GENERAL PUBLIC UTILITIES SERVICE CORPORATION ON
JUNE 11, 1975

On June 11, 1975, a meeting was held in Bethesda, Maryland with General Public Utilities Service Corporation (GPUSC) to discuss the status of the reevaluation of single electrical failures that may affect the ECCS at the Oyster Creek Nuclear Generating Station (OCNGS). This meeting was requested by GPUSC. A list of attendees is enclosed.

Background

On May 24, 1975, we issued a safety evaluation of the reanalysis of the Oyster Creek ECCS cooling performance submitted by Jersey Central Power and Light Company. A condition was added to the Provisional Operating License based on our review which requires the licensee to perform a complete reassessment of all elements of the electrical systems associated with ECCS performance to verify that no single passive electrical failure would adversely affect the ability of the ECCS to conform to the evaluation submitted to demonstrate compliance with 10 CFR Section 50.46.

Summary of Meeting

The single failure reassessment is not yet complete. The following assumptions are being used by GPUSC:

1. If a break occurs in a cooling system, damage from pipe whip or jet impingement is not taken into account (except it is assumed that sensors attached to the broken line do fail).
2. Complete electrical cabinet or panel failures are not taken into account.
3. Fires, earthquakes, or natural phenomena are not considered.
4. There are no environmental considerations taken into account other than those considered in the original design.
5. Physical separation, other than what was included in the original design, is not considered.



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6. Credit is taken for circuit breakers being fault-limiting devices (e.g.: no failure of auto-bus transfer).

Considering the above assumptions, GPUSC stated that the investigation to date has identified the following potential single failures in electrical systems:

1. Loss of power to DC1 will not permit opening of the emergency condenser valves. The effect of this loss of emergency condenser capability would increase the peak clad temperature about 10°F.
2. The loss of "A" recirculation loop as a result of a recirculation line break plus a single passive electrical failure in the "E" loop would eliminate the availability of the emergency condensers. An increase in the peak clad temperature of about 10°F would result.
3. Loss of DC power to panel "D" or "E" would prevent automatic actuation of the Automatic Depressurization System (ADS). This occurrence would result in an increase of a few hundred degrees in the peak clad temperature. Manual actuation of the ADS can be accomplished even if DC power is lost.
4. A hot short to ground could cause the manual control valves to close in the core spray system.
5. Failure of an under-voltage relay to open on loss-of-bus voltage could result in the loss of a core spray system.

The final report of the single failure reevaluation is due to be submitted to us for our evaluation on or before June 24, 1975.

Walter A. Paulson

Walter A. Paulson
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosure:
Attendance List

cc: See next page

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ENCLOSURE

OYSTER CREEK NUCLEAR

GENERATING STATION

SINGLE FAILURES

Nuclear Regulatory Commission

W. A. Paulson
R. Woods
R. Baer
C. Miller
G. Lear

General Public Utilities Service Corporation

T. Crimmins
L. Lanese
N. Trikouros

NUS

G. C. Reeder

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DISTRIBUTION OF MEETING SUMMARY

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