

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION I

IE Inspection Report No: 50-219/75-04

Docket No: 50-219

Licensee: Jersey Central Power and Light Company

License No: DPR-16

Madison Avenue at Punch Bowl Road

Priority: _____

Morristown, New Jersey 07960

Category: C

Location: Oyster Creek, Forked River, New Jersey

Safeguards Group: _____

Type of Licensee: BWR (G.E.) 1930 MW(t)

Type of Inspection: Routine, Unannounced

Dates of Inspection: January 29-31, 1975

Dates of Previous Inspection: January 14-15, 17 and 20, 1975

Reporting Inspector: *D.L. Capton for*
E. G. Greenman, Reactor Inspector

2/26/75
Date

Accompanying Inspectors: NONE

Date

Date

Date

Date

Other Accompanying Personnel: NONE

Date

Reviewed By: *D.L. Capton*
D. L. Capton, Senior Reactor Inspector
IE:I Operations Branch

2/26/75
Date

B/881

SUMMARY OF FINDINGS

Enforcement Action

A. Violations

None

B. Infractions

Contrary to Technical Specification Table 3.1.1.H.2, Isolation condenser high flow line break sensor 1B11A2 actuated in excess of 27 inches water during surveillance. (JCP&L letter to Division of Reactor Licensing dated December 2, 1974, Subject AO 74-60).

C. Deficiencies

Contrary to Technical Specifications 6.1.C.1.g and Administrative Procedure 102.8 "Standard Orders for Operations Personnel", Revision 1 dated August 19, 1969 the status and applicability of all standing orders was not reviewed quarterly by the Plant Operations Review Committee. (Details 2d)

Licensee Action on Previously Identified Enforcement Items

1. Failure to distribute RWP to job location - Licensee is revising station procedures to be more explicit. Item remains unresolved pending procedural revision.
2. Failure to maintain average rate of reactor coolant temperature change with 100°F during a one hour period, during a cooldown - Licensee has issued data sheets for plotting temperature heatup and cooldown and instructions to operating personnel. Item remains unresolved pending incorporation of requirements in operating procedures.

Design Changes

Not Inspected

Unusual Occurrences

The following abnormal occurrences and events were reviewed. Comments concerning specific areas are noted within this report.

- A. Failure of isolation condenser condensate high flow line break sensor 1B11A2 to actuate within T.S. limits during surveillance.¹
- B. Failure of one reactor building to torus vacuum breaker valve to demonstrate operability during quarterly surveillance testing.²
- C. A spent fuel shipment arrived at NFS, West Valley, N.Y. with reported cask external contamination levels on one smear of approximately 22,000 dpm. (Details 7)

Other Significant Findings

A. Current Findings

1. Acceptable Areas

a. Plant Operations

An audit of plant operations was conducted. (Details 3 and 4)

b. Organization and Administration

An audit of PORC and GORB activities as delineated in Section 6 of the Technical Specifications was conducted. (Details 2)

2. Unresolved Items

- a. Verification of GORB Committee qualifications. (Details 2c)

B. Status of Previously Reported Unresolved Items

- 1. Development and approval of procedures for relay testing (IE Inspection Report 73-18). Item is considered resolved. (Details 6a)

1. JCP&L Report to IE:I dated November 25, 1974, Subject AO 74-60
2. JCP&L Report to IE:I dated January 9, 1975, Subject AO 75-1

Management Interview

A management interview was conducted on January 31, 1975 with Mr. J. T. Carroll, Station Superintendent, Mr. D. Reeves, Chief Engineer and Mr. J. Menning, Staff Engineer. Mr. D. A. Ross, Manager, Nuclear Generating Stations participated via telephone intercom. Items discussed are summarized below:

A. General

The inspector summarized the scope of the routine unannounced inspection relative to a review of operating logs and records, PORC meeting minutes, GORB audit results, GORB meeting minutes, verifications of on and off-site committee memberships versus Technical Specification requirements, a facility tour including observations of snubber inspections in progress January 30, 1975 and review of abnormal occurrences since the last inspection. The inspector also noted he had continued a review of ongoing spent fuel shipments from the site.

B. Status of Previously Identified Enforcement Action

1. RWP Distribution Requirements (IE Inspection 50-219/74-16 dated November 22, 1974)

The inspector stated that he had reviewed the Radiation Safety Manual, and had no further questions regarding this matter at this time. The inspector further stated that the subject of RWP posting was considered unresolved pending formal procedural revision.

2. Cooldown Limits (IE Inspection 50-219/74-18 dated December 26, 1974)

The inspector stated that he had reviewed data sheets prepared by the licensee for plotting reactor coolant temperature heat-up and cooldown and instructions and had no further questions regarding this matter at this time. The inspector further stated that this item was considered unresolved pending incorporation of instructions into applicable operating procedures.

A licensee representative acknowledged the inspector's statements.

C. Control Room and Shift Foreman Logs

The inspector stated that his review of the referenced logs indicated that more emphasis needed to be placed on entry completeness. Shift entries regarding assumption of duties and Shift Foreman entries were used as specific examples.

A licensee representative concurred with the inspector's comments and stated that appropriate emphasis would be provided to the operating staff. (Details 3)

D. Administrative Procedures

The inspector stated his position that the revised administrative procedures already written should be issued in a timely fashion to preclude procedural violations of the type identified during this inspection and enhance document control.

A licensee representative acknowledged the inspector's position and referenced the interface between the Administrative Procedures and Operating Procedures undergoing revision. (Details 2f)

E. GORB Qualifications

The inspector stated that resumes of GORB members were not available for on-site review to verify compliance with Technical Specification requirements.

A licensee representative stated that this information would be made available for on-site review. (Details 2c)

F. Enforcement Action

The item listed under enforcement action above was identified as an apparent deficiency. (Details 2a)

DETAILS

1. Persons Contacted

Mr. D. A. Ross, Manager, Nuclear Generating Stations
Mr. J. T. Carroll, Station Superintendent
Mr. D. L. Reeves, Chief Engineer
Mr. E. I. Riggle, Maintenance Supervisor
Mr. J. P. Maloney, Operations Supervisor
Mr. K. O. Fickeissen, Technical Supervisor
Mr. E. D. Skalsky, Radiation Protection Supervisor
Mr. J. Menning, Staff Engineer

2. Administration and Organization

a. Plant Operations Review Committee Meetings

The PORC met on the following dates and the minutes were reviewed by the inspector. The inspector verified that committee meetings and actions were consistent with Technical Specification requirements except as noted.

<u>Meeting No.</u>	<u>Date</u>
79-74	October 25, 1974
80-74	October 28, 1974
81-74	October 31, 1974
82-74	November 12, 1974
83-74	November 14, 1974
84-74	November 18, 1974
85-74	November 25-26, 1974
86-74	November 27, 1974
87-74	December 4, 1974
88-74	December 12, 1974
89-74	December 20, 1974
90-74	December 23, 1974
91-74	December 29, 1974

The inspectors review indicated that PORC had not reviewed the status and applicability of standing orders as stated in Administrative Procedure No. 102.8, Revision 1 dated August 19, 1969. (DEFICIENCY)

b. General Office Review Board Meetings

The GORB met on the following dates and the minutes were reviewed by the inspector. The inspector verified that committee meetings and actions were consistent with Technical Specification requirements.

<u>Meeting No.</u>	<u>Date</u>
48A	January 18, 1974
49	April 17, 1974
50	March 19, 1974
50	March 20, 1974
50A	March 29, 1974
50B	April 26, 1974
51	May 16, 1974
51	May 17, 1974
52	June 13, 1974
53	August 6, 1974
53A	August 19, 1974

c. GORB Membership Qualifications

Data was not available on site to demonstrate that members of GORB, with regard to combined experience and technical specialties met requirements of the Technical Specifications. The inspector advised a cognizant licensee representative that this information should be provided on site. This item is considered outstanding and was discussed at the exit interview.

d. Standing Orders and Required Reading Files*

The inspector reviewed Standing Orders and Required Reading. Updating had been completed and no obsolete material was maintained. Facility changes and revised procedures were current. This item is considered resolved.

e. Shift Turnover Procedures**

The inspector reviewed Administrative Procedure Section 103, Revision 3 dated April 11, 1974. Section 103.12.4-2 has been revised to include planned and or continuing maintenance activities that directly affect plant operation. This item is considered resolved.

* IE Inspection Report 50-219/74-02, Details 7.b

** IE Inspection Report 50-219/74-03, Details 4, dated April'26, 1974

f. Administrative Procedures

The licensee is rewriting facility procedures to achieve conformance with ANSI 18.7 and Regulatory Guide 1.33. New Administrative Procedures have been written. Implementation has not been completed. This item was discussed at the exit interview and remains outstanding.

3. Logs and Records

The following logs and records were reviewed without comment except as noted within this report.

- a. PORC meeting minutes - October 25 - December 29, 1974
- b. GORB meeting minutes - January 18 - August 18, 1974
- c. GORB audits - March - June 26, 1974
- d. Shift Foremans Log - November 8, 1974 - January 26, 1975
- e. Control Room Log - November 8, 1974 - January 26, 1975
- f. Standing Orders Log - No. 1-20
- g. Daily TS Logs - November 1, 1974 - January 26, 1975
- h. Reactor Building Tour Check Sheets - November 1, 1974 -
January 26, 1975
- i. Turbine Building Tour Check Sheets - November 1, 1974 -
January 26, 1975
- j. Rod Waste Building Tour Check Sheets - November 1, 1974 -
January 26, 1975

The inspector's review of Control Room and Shift Foreman Logs indicated non-uniformity in entries of different shifts. Specific inconsistencies were discussed at the exit interview. This area remains outstanding.

4. Operations

The inspector reviewed operating logs and records (Details 3) and held discussions with operating personnel. A tour of the facility was conducted and the inspector also observed ongoing seismic shock suppressor inspections in the reactor building. Logs and licensee actions appeared consistent with Technical Specification requirements.

5. Primary Coolant System

a. Disparity Between Recirculation Loop and Multipoint Recorder Temperature Indications*

The licensee has issued instructions regarding log book entries to require an entry accompanying start of a recirculation pump and including differential temperature between the reactor coolant and the idle recirculation loop. The differential temperature is determined by subtracting the temperature of the idle loop from the temperature of operating loops, as determined from the recirculation pump suction temperature recorder. This item is considered resolved.

6. Emergency Power

a. Relay Procedure Development**

The inspector reviewed the licensee's Relay Check List and Test Procedure dated May 23, 1974, and procedural specifications concerning testing of differential monitoring relays. Procedural requirements included a return of all taps to position as indicated on relay setting sheets and requests for relay testing through the cognizant Shift Foreman. This item is considered resolved.

b. Standby Diesel Generator Fuel Tank Level

The inspector verified from daily TS logs for the period November 1, 1974 - January 26, 1975 that minimum levels established by the TS were maintained. The minimum level recorded was 14,878 gallons of diesel fuel.

7. Fuel Handling

a. Spent Fuel Shipment

The inspector reviewed the licensee's radioactive materials shipment record for shipment No. OC-30-75 dated January 28, 1975, and associated smear survey results involving approximately 100 smears and 3 decontaminations of the NFS.4 cask.

* IE Inspection Report 50-219/74-02, Details 5, dated March 27, 1974

** IE Inspection Report 50-219/73-15, Details 5, dated October 23, 1973

The maximum contamination on the outer surface of the NFS-4 container was measured by the licensee to be 1,480 dpm-beta. Cask arrival was reported by NFS with one smear indicating 22,000 dpm. Location had not been determined during the inspector's review. The licensee was developing test data for a shipment scheduled for January 31, 1975 to provide data comparisons. The inspector identified no unresolved or noncompliance items as a result of review of the licensee's data or techniques. This item will be reviewed further during subsequent shipments.