NRC Form 366
(9/83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	0 5 0 0 0 3 6 1	1 OF 0 2

TITLE (4)
GAS STRIPPER LEAKAGE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 9	2 4	8 4	8 4	— 0 5 4	— 0 0	1 0	2 4	8 4	SONGS Unit 3	0 5 0 0 0 3 6 2
										0 5 0 0 0

OPERATING MODE (9)	1
POWER LEVEL (10)	1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
	20.405(a)(1)(i)		50.36(c)(1)	X	50.73(a)(2)(v)		73.71(c)
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)		50.73(a)(2)(i)	X	50.73(a)(2)(viii)(A)		
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
J. G. HAYNES, STATION MANAGER	7 1 4 — 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	W E	V	G 2 5 5	N							

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9/24/84, at 1610, with Units 2 and 3 in Mode 1 at 100% power, the low level alarms were received on Plant Ventilation Stack Monitors 2RE-7865 and 3RE-7865. The monitor levels increased to a peak of 9.5E-4 uCi/cc on 2RE-7865. Actions were immediately taken to terminate the release. The release was terminated at 1632 when the gaseous radwaste processing gas strippers were bypassed. The source of the release was later traced to the south gas stripper.

The cause of the release was determined to be the failure of South Gas Stripper Preheater Drain Valve SA1901MU702 which allowed a mixture of Unit 3 pressurizer degas effluent and T067 and T068 recirc to escape via the floor drains into the Radwaste Building Ventilation System which exhausts to the Plant Vent Stack.

The release was approximately 117.5 curies of Xe-133. The concentration in unrestricted areas, when averaged over one hour, was 7.8E-7 uCi/cc (2.6 times the applicable concentration in Appendix B, Table II of 10 CFR 20). The release was within Technical Specification limits.

Gas Stripper Preheater Drain Valve SA1901MU702 was repaired, successfully tested, and returned to service.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQ. NUMBER	REV. NUMBER	
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	0 5 0 0 0 3 6 1	8 4	0 5 4	0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 24, 1984, at 1610, with Units 2 and 3 in Mode 1 at 100% power, the low level alarms were received on Plant Ventilation Stack Monitors (EIIS Component Code V) 2RE-7865 and 3RE-7865. The monitor levels increased to a peak of 9.5E-4 uCi/cc on 2RE-7865. Actions were immediately taken to terminate the release. The release was terminated at 1632 when the gaseous radwaste processing gas strippers were bypassed. The source of the release was later traced to the south gas stripper.

The cause of the release was determined to be the failure of South Gas Stripper Preheater Drain Valve SA1901MU702 (EIIS Component Code V) which allowed a mixture of Unit 3 pressurizer degas effluent and Primary Tanks T067 and T068 recirc to escape via the floor drains into the Radwaste Building Ventilation System (EIIS System Code VH) which exhausts to the Plant Vent Stack (EIIS System Code VL). Gas Stripper Preheater Drain Valve SA1901MU702 was repaired, successfully tested, and returned to service.

The release was approximately 117.5 curies of Xe-133. The concentration in unrestricted areas, when averaged over one hour, was 7.8E-7 uCi/cc (2.6 times the applicable concentration in Appendix B, Table II of 10 CFR 20). The release was within Technical Specification limits.

After the alarms and release were terminated, the quantitative assessments of the release were delinquent, as to whether Technical Specification release limits and/or 10 CFR 50.72 reporting requirements were involved. This assessment was delinquent because the Shift Superintendent did not promptly assure Chemistry had been contacted to perform the release calculations. Because of this delay in performing the release calculations, although the assessment showed no Technical Specifications were exceeded, the 10 CFR 50.72(b)(2)(iv)(a) report, made at 2327, was delinquent.

Special Order 84-25, "Evaluation of Gaseous Effluent Monitor Alarms," was issued to reinforce policy regarding the necessity and importance of timeliness and complete follow-through in response to gaseous effluent monitor alarms. The alarm response procedures, Operating Instruction S023-0-25, "Telephone Notification of the NRC for Significant Events," and the Emergency Plan Implementing Procedures were reviewed and determined to be adequate. Due to the error on the part of the Shift Superintendent, the individual involved received a written reprimand.

There are no credible circumstances that could have increased the severity of this event.

Southern California Edison Company

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SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

October 24, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 84-054
San Onofre Nuclear Generating Station, Units 2 and 3

Pursuant to 10 CFR 50.73(a)(2)(v) and 50.73(a)(2)(viii), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Waste Gas Processing System. Since this occurrence involved a shared system between Units 2 and 3, a single report is enclosed in accordance with NUREG-1022. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely

JGHaynes

Enclosure: LER No. 84-054

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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