

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 5	PAGE (3) 1 OF 0 1
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TITLE (4) Automatic Reactor Scram Resulting From Main Condenser Low Vacuum Trip of the Unit Main Turbine

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 1	2 4	8 5	8 5	0 0 8	0 0	0 2	2 2	8 5			0 5 0 0 0
											0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) 1	20.402(b)	20.406(e)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 3 5	20.406(a)(1)(i)	50.38(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(e)
	20.406(a)(1)(ii)	50.38(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME M. J. Pastva, Jr., Regulatory Technician	TELEPHONE NUMBER AREA CODE 9 1 9 4 5 7 1 2 3 1 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS
X	W/F	R/LY	T 2 7 4	No					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 1/24/85, at 2312, a Unit 1 reactor scram automatically occurred due to a main turbine trip and stop valve closure resulting from a low vacuum in the unit main condenser. Unit 1 was at reactor power level of 37%. A controlled decrease in reactor power was in progress in response to a steadily decreasing main condenser vacuum.

During the unit scram recovery, the lowest reactor level recorded value was 153". This was controlled by use of the Reactor Condensate System. Primary containment group isolations 2, 6, and 8 occurred.

The decreasing main condenser vacuum resulted from a buildup of demineralized water in the Unit 1 off-gas filter and the Units 1 and 2 common standby off-gas filter. The high moisture on these filters created a back pressure due to the reduced ability to pass the off-gas flow. A defective latching relay, 1-OG-LY-4910, in the unit off-gas piping loop seal reservoir level control circuitry, caused the reservoir solenoid-operated makeup valve, 1-OG-SV-4906, to remain open. Overflow of the reservoir into the unit off-gas piping and filters resulted.

The latching relay, Transamerica Delaval Part No. ST-28196, was replaced to reestablish proper level control of the loop seal reservoir. The Unit 1 off-gas filter and Units 1 and 2 common standby off-gas filter were replaced.

The event occurred in the most limiting plant condition.

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PDR ADOCK 05000325
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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
February 22, 1985

FILE: B09-13510C
SERIAL: BSEP/85-0295

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-008

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/dgr/LETDR1

Enclosure

cc: Dr. J. N. Grace

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11