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Harris Nuclear Plant
5413 Shearon Harris Road
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10 CFR 50.90

April 16, 2020
Serial: RA-20-0105

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Shearon Harris Nuclear Power Plant, Unit 1
Docket No. 50-400
Renewed License No. NPF-63

Subject: Supplement to License Amendment Request to Revise Technical Specifications for the Adoption of 10 CFR 50, Appendix J, Option B for Type B and C Testing and for Permanent Extension of Type A, B and C Leak Rate Test Frequencies

Ladies and Gentlemen:

By letter dated November 8, 2019 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML19312C931), Duke Energy Progress, LLC (Duke Energy) submitted a license amendment request (LAR) for the Shearon Harris Nuclear Power Plant, Unit 1 (HNP). The proposed amendment would modify Technical Specification (TS) requirements to allow extension of the Type A integrated leakage rate test program test interval from 10 years to 15 years, and an extension of the containment isolation valve leakage rate testing frequency for Type C leakage rate testing of selected components to a maximum of 75 months. The LAR also proposes to adopt Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix J, Option B, as modified by approved exemptions, for the performance-based testing of Type B and C tested components and the use of American National Standards Institute/American Nuclear Society (ANSI/ANS) 56.8-2002, "Containment System Leakage Testing Requirements."

Attachment 1 of this supplemental correspondence provides additional markups to TS 3.6.1.3, "Containment Air Locks," to address editorial items identified related to the proposed markups for TS 3.6.1.2, "Containment Leakage," provided in Attachment 2 of the original submittal. Specifically, it addresses the removal of the word "overall" from Note 3 and Action c.1 of TS 3.6.1.3, as consistent with the proposed revision to TS 3.6.1.2 in the original submittal. It also adjusts Note 3 of TS 3.6.1.3 to refer to "Specification 3.6.1.2" instead of "Specification 3.6.1.2.a" for the containment leakage rate, consistent with the proposed revision to TS 3.6.1.2 provided in the original submittal.

The content of this supplemental correspondence does not change the scope nor the No Significant Hazards Consideration provided in the original submittal. This correspondence also does not contain any regulatory commitments.

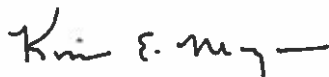
In accordance with 10 CFR 50.91, Duke Energy is providing the state of North Carolina with a copy of this supplemental correspondence.

Please refer any questions regarding this submittal to Art Zaremba, Manager – Nuclear Fleet Licensing, at (980) 373-2062.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on April 16, 2020.

Sincerely,



Kim Maza
Site Vice President
Harris Nuclear Plant

Attachment:

1. Additional Proposed Technical Specification Changes

cc: J. Zeiler, NRC Sr. Resident Inspector, HNP
W. L. Cox, III, Section Chief, N.C. DHSR
T. Hood, NRC Project Manager, HNP
L. Dudes, NRC Regional Administrator, Region II

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U.S. Nuclear Regulatory Commission
Serial: RA-20-0105
Attachment 1

ATTACHMENT 1

ADDITIONAL PROPOSED TECHNICAL SPECIFICATION CHANGES

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-400

RENEWED LICENSE NUMBER NPF-63

3 PAGES (INCLUDING COVER)

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Two containment air locks shall be OPERABLE:

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

..... Notes

1. Entry and exit is permissible to perform repairs on the affected air lock components.
2. A separate ACTION is allowed for each air lock.
3. Enter 3.6.1.1 LCO for "Containment Integrity" when the air lock leakage results in exceeding the ~~overall~~ containment leakage rate, Specification 3.6.1.2.a.
4. Locking a Personnel Air Lock door shut consists of locking the associated manual pumping stations and deactivating the electronic mechanisms used to open a Personnel Air Lock door once the associated air lock door is shut. Locking an Emergency Air Lock door shut consists of locking the mechanical operator.

.....

- a. One or more containment air locks with one containment air lock door inoperable:#
 1. Within one hour, verify the OPERABLE door is closed in the affected air lock, and
 2. Within 24 hours, lock the OPERABLE door closed in the affected air lock, and
 3. Once per 31 days, verify the OPERABLE door is locked closed in the affected air lock*, or
 4. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

1. ACTIONS 3.6.1.3.a.1, 3.6.1.3.a.2, 3.6.1.3.a.3, and 3.6.1.3.a.4 are not applicable if both doors in the same air lock are inoperable and ACTION 3.6.1.3.c is entered.

2. Entry and exit is permissible for 7 days under administrative controls if both air locks are inoperable.

* Air lock doors in high radiation areas may be verified closed by administrative means.

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

- b. One or more containment air locks with containment air lock interlock mechanism inoperable.##
1. Within one hour, verify an OPERABLE door is closed in the affected air lock, and
 2. Within 24 hours, lock an OPERABLE door closed in the affected air lock, and
 3. Once per 31 days, verify the OPERABLE door is locked closed in the affected air lock*, or
 4. Otherwise, be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. One or more containment air locks inoperable for reasons other than 3.6.1.3.a or 3.6.1.3.b.
1. Immediately initiate action to evaluate ~~overall~~ containment leakage rate per LCO 3.6.1.2, and
 2. Within one hour, verify a door is closed in the affected air lock, and
 3. Within 24 hours, restore air lock to OPERABLE status, or
 4. Otherwise be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

1. ACTIONS 3.6.1.3.b.1, 3.6.1.3.b.2, 3.6.1.3.b.3, and 3.6.1.3.b.4 are not applicable if both doors in the same air lock are inoperable and ACTION 3.6.1.3.c is entered.

2. Entry and exit of containment is permissible under the control of a dedicated individual.

* Air lock doors in high radiation areas may be verified closed by administrative means.