U.S. NUCLEAR REGULATORY COMMISSION

REGION I

Report No. 50-286/84-22

Docket No. 50-286

License No. DPR-64

Priority --

Category C

Licensee: Power Authority of the State of New York 10 Columbus Circle New York, New York 10019

Facility Name: Indian Point Nuclear Generating Station, Unit 3 Inspection conducted: September 16, 1984 to October 15, 1984 Inspectors:

P. S. Koltay, Senfor Resident Inspector

Rossbach, Resident Inspector

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10/25/84

Approved by:

Leif Norrholm, Chief, Reactor Project Section 2B, DPRP

Inspection Summary:

Inspection on September 16, 1984 to October 15, 1984 (Inspection Report 50-286/84-22)

Areas Inspected: Routine onsite regular and backshift inspection of plant operations including shift logs and records; operational safety verification; maintenance; surveillance; review of monthly report; licensee event reports; generic letter followup; and, security finding. The inspection involved 144 inspector hours by the resident inspectors.

<u>Results</u>: One violation was identified involving improper storage of safeguards information. The licensee took prompt corrective action. The unit began a mid-cycle outage to inspect the steam generators at the end of this inspection period.

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PETAILS

1. Persons Contacted

Within this report period, interviews and discussions were conducted with members of the licensee management and staff to obtain the necessary information pertinent to the subjects being inspected.

2. Operational Safety Verification

A. Documents Reviewed:

- Selected Operators' Logs
- Shift Supervisors Log
- Selected Shift Turnover Checklists
- Jumper Log
- Radioactive Waste Release Permits (liquid & gaseous)
- Selected Radiation Exposure Authorizations (REA's)
- Selected Chemistry Logs
- Selected Tagouts
- Health Physics Watch Log
- B. The inspector(s) conducted routine entries into the protected area of the plant, including the control room, PAB, fuel building, and containment (when access is possible.) During the inspection activities, discussions were held with operators, technicians (HP & I&C), mechanics, foremen, supervisors, and plant management. The purpose of the inspection was to affirm the licensee's commitments and compliance with 10 CFR, Technical Specifications, and Administrative Procedures.
 - On a daily basis, particular attention was directed in the following areas:
 - Instrumentation and recorder traces for abnormalities;
 - Adherence to LCO's directly observable from the control room;
 - Proper control room and shift manning and access control;
 - Verification of the status of control room annunciators that are in alarm;
 - Proper use of procedures;
 - Review of logs to obtain plant conditions; and,
 - Verification of surveillance testing for timely completion.

- On a weekly basis, the inspector(s) confirmed the operability of a selected ESF train by:
 - Verifying that accessible valves in the flow path were in the correct positions;
 - Verifying that power supplies and breakers were in the correct positions;
 - Verifying that de-energized portions of these systems were de-energized as identified by Technical Specifications;
 - Visually inspecting major components for leakage, lubrication, vibration, cooling water supply, and general operable condition; and,
 - Visually inspecting instrumentation, where possible, for proper operability.

Systems Inspected:

- Safety Injection
- Containment Spray
- Emergency Boration
- Auxiliary Feedwater
- 3. On a bi-weekly basis, the inspector(s):
 - Verified the correct application of a tagout to a safety related system;
 - Observed a shift turnover;
 - Reviewed the sampling program including the liquid and gaseous effluents;
 - Verified that radiation protection and controls were properly established;
 - Verified that the physical security plan was being implemented;
 - Reviewed licensee-identified problem areas; and,
 - Verified selected portions of containment isolation lineup.

C. Inspector Comments/Findings:

The unit operated at 100% power, except as delineated below, during this inspection period. The inspector monitored selected phases of the unit's operation, and determined that the areas inspected did not constitute a health and safety hazard to the public or plant personnel.

September 17

At 11:45 a.m., a fuse blew in the safety injection rack for train #1, and resulted in the loss of the ability to automatically initiate engineered safety features in train #1. The fuse blew due to a short in a light socket in the rack. At 12:00 nocn, repairs were completed. (See section 3 of this report for further details). A retest was also performed. All of the engineered safety features affected could have been started manually from the control room except for one emergency diesel. That diesel would have started automatically on undervoltage and could be manually started at the diesel. The other two diesels would have been started automatically by a SI signal from train #2 which was unaffected by this event. The unit was operating at 100% throughout this event.

- October 6 At about 10 A.M., A 40 MWe turbine runback occurred while putting #33 reactor coolant loop instrumentation in bypass in preparation for repairing a loop flow transmitter. The runback was caused by positioning a switch out of sequence while bypassing this instrumentation. As a result of this runback, a Temporary Procedure Change was issued to Alarm Response Procedure #9 to clarify operator actions to the "RTD Bypass Low Flow" alarm.
- October 12 Power reduction was begun at about 5:00 p.m. on October 12 and the unit was placed in cold shutdown at 4:15 p.m. on October 13, thus beginning a midcycle outage as discussed in the following paragraph.

D. Mid-Cycle Steam Generator Inspection Outage

Prior to startup from a 1982 to 1983 outage during which extensive steam generator tube and girth weld repairs were completed, the licensee committed to reinspect the steam generator tubes and girth welds during the middle of this operating cycle. At the end of this inspection period, the plant was in cold shutdown and the licensee was degassing the primary system in preparation for the inspections. During this outage, the licensee will replace seal packages on two reactor coolant pumps, replace electrical equipment in accordance with the environmental qualifications program, and also work on various maintenance items, surveillances and modifications. The resident inspectors will follow this work.

On September 13, 1984, the licensee informed the NRC that based on a review of eddy current data and tube plugging records in preparation for this outage, the licensee discovered that two tubes in #31 steam generator had degradation, due to pitting beyond the plugging limit of 50%, but had not been plugged. The maximum defect for each tube was 54%. The licensee requested a temporary Technical Specification change through a technical specification amendment to raise the plugging limit on these two tubes to 55% until this outage, when they will be plugged. The licensed reactor operators reviewed Off Normal Operating Procedures on steam generator tube leakage as requested in NRC's review of this amendment request. There was no indication of any steam generator tube leakage during this operating cycle.

No violations were identified.

3. Maintenance

- A. The inspector selected completed maintenance activities listed below to ascertain the following:
 - The activities did not violate a limiting condition for operation;
 - That redundant components were operable;
 - That equipment was tagged out in accordance with licensee approved procedures;
 - That approved procedures, adequate to control the activity, were being used by qualified technicians;
 - That Q/C hold points were observed and that materials were properly certified;

- That radiological controls were proper and in accordance with licensee approved radiation exposure authorization; and,
- That the equipment was properly tested prior to return to service.
- 1) Inspect and PM Fan Cooler Unit Breaker

Documents Reviewed:

- Work Request 5205
- Work Sheet
- Work Procedure PM-R-ES-6
- 2) #33 Fan Cooler Unit Motor Repair

Documents Reviewed:

- Work Request 5190
- Work Sheet and Steplists
- Request for Material Substitution
- Motor Certification
- Retest

3) SI Logic Cabinet Light Socket Short

Documents Reviewed:

- Work Request 3163
- Work Sheet and Checklist
- Light Socket Certification
- Retest
- 4) #43 Power Range Detector Power Supply Replaced

Documents Reviewed ·

- Work Request 3094
- Work Sheet and Checklist
- Power Supply Certification

No violations were identified.

4. Surveillance

A. Documents Reviewed:

	3PT-M6A	6.9KV Undervoltage Analog Channel Functional Test
-	3PT-BW3	Inspect for Service Water Leaks in Containment
-	3PT-M19	Auxiliary Component Cooling Pump Functional Test
-	3PT-TM02	SOP-CB-1, Containment Integrity

B. Inspector Findings:

The inspector(s) directly observed the performance of portions of the above-listed tests, or reviewed completed surveillance procedures to ascertain the following:

- That the instrumentation used was properly calibrated;
- That the redundant system or component was operable, where required;
- That properly approved procedures were used by qualified personnel;
- That the acceptance criteria were met;
- That proper reviews, by the licensee, had been conducted; and.
- That the results of the tests met Technical Specification requirements.

The inspector(s) also verified that the systems were properly returned to service following the above-listed tests, by observing actual valve and switch positions or position indication in the control room.

No violations were identified.

5. Review of Monthly Report

The Monthly Operating Report for August, 1984 was reviewed. The review included an examination of selected maintenance work requests, and an examination of significant occurrence reports to ascertain that the summary of operating experience was properly documented.

The inspector(s) verified through record reviews and observations of maintenance in progress that:

- The corrective action was adequate for resolution of the identified item; and,
- The operating report included the requirements of TS 6.9.1.5.

The inspector(s) have no further questions relating to the report.

6. Licensee Event Reports

A. In-Office Review of Licensee Event Reports

The inspectors reviewed LER's submitted to the NRC:RI office to verify that details of the event were clearly reported, including the accuracy of the description of cause and adequacy of corrective action. The inspector determined whether further information was required from the licensee, whether generic implications were involved, and whether the event warranted onsite followup.

The following LER was reviewed:

- 84-013 Unit Trip (Feedwater Transient)
- B. Onsite Licensee Event Followup

The LER listed above was reviewed to verify that the reporting requirements of Technical Specifications and Station Administrative Procedures had been met, that appropriate corrective action had been taken, that the event was reviewed by the PORC (Plant Operating Review Committee), and that continued operation of the facility was in conformance with the Technical Specification limits. This event was reviewed and documented in Report 84-16.

No violations were identified.

- 7. Generic Letter 83-28 (Salem ATWS Events) Followup
 - A. Documents Reviewed:
 - Licensee letters datcd September 8, 1983, November 7, 1983;
 - AP-21.2, Post Trip Review/Restart Procedure
 - AP 8, Reporting of Significant Occurrences;
 - Several post trip reviews
 - B. The inspector(s) reviewed the licensee's implementation of their response to Generic Letter 83-28 in the area of post-trip reviews. The inspector(s) verified that a post-trip review program has been implemented and that:
 - Procedures require safety reviews of reactor trips:
 - Post-trip review procedures are reviewed periodically and upgraded;

- Training is given in post-trip review procedures;
- Responsibilities and authorities for performing the post-trip review and authorizing restart are defined;
- Criteria for comparing plant information with essential plant behavior have been established;
- Information and data systems exist to support posttrip reviews; and,
- Post-trip review data and records are retained.

No violations were identified.

8. Storage of Safeguards Information

On September 21, while reviewing controlled drawings storage facility, in the Instrumentation and Controls (I&C) Work Shop located in the Administration Building, the inspector(s) identified approximately 30 drawings marked "Safeguards Information." The inspector(s) verified that the subject drawings contained wiring diagrams of various security alarm systems and consoles and wore stored in an unsecured cabinet in an area which is not required to be locked or continuously attended. Licensee personnel in the Instrumentation and Control Department are authorized to have access to such safeguards information on a "need to know" basis, in order to facilitate equipment testing and maintenance. However, when such documents are not in use, the guidelines for proper storage arrangements outlined in the licensee's procedure, SPD 4.1, "Protection of Safeguards Information", Revision 0 must be met.

The above procedure encompasses the requirements of 10 CFR 73.21, "Requirements for the Protection of Safeguards Information." Section 2.3 of the procedures identifies wiring diagram of alarm assessment equipment and alarm systems as safeguards information. Section 11.1 of the procedure requires unattended safeguards information to be stored in a locked security storage containers.

The licensee's failure to control the storage of safeguards information in accordance with procedural requirements constitutes a violation. (84-22-01)

The above item was brought to the licensee's attention. As an immediate corrective action, the licensee moved the drawings to a properly secured storage cabinet. Subsequently, the licensee installed a lock on the original cabinets and returned the drawing to the I&C workshop. In addition, the licensee also installed locks on the two drawers located in the document vaults containing physical security drawings thus restricting access to personnel who have a "need to know", and are authorized to have access to safeguards information.

9. Site Visit by NRC Commissioners

The Indian Point 3 site was visited by Commissioner Lando W. Zech, Jr. on September 26, and by Commissioner James K. Asselstine on September 28. On each occasion, the Commissioner met with licensee's onsite and offsite management to discuss plant status and applicable current issues. The resident inspectors accompanied each Commissioner on a plant tour. The visits concluded with a short news conference attended by local news reporters. Several intervenors accompanied the Commissioners during the visits.

10. Exit Interview

At periodic intervals during the course of the inspection, meetings were held with senior facility management to discuss the inspection scope and findings. An exit interview was held on October 15, 1984 to discuss this report period. During the discussion, the licensee did not identify any 10 CFR 2.790 material.