

### LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>D. C. COOK NUCLEAR PLANT, UNIT 1</b>	DOCKET NUMBER (2) <b>0 5 0 0 0 3 1 1 5 1</b>	PAGE (3) <b>1 OF 0 2</b>
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TITLE (4)  
**FAILURE TO COMPLY WITH THE ACTION STATEMENT OF TECHNICAL SPECIFICATION 3.8.2.4.**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	1	1 8 8	5	8 5	0 0 3	0	2	1 4 8 5			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) <b>5</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)									
POWER LEVEL (10) <b>0 0 0</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME <b>K. R. BAKER, OPERATIONS SUPERINTENDENT</b>	TELEPHONE NUMBER <b>6 1 6 4 6 5 - 5 9 0 1</b>
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS
B	E J B	T R Y	E 3 5 5	N					

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (if yes complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR 
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ABSTRACT (Limit to 1400 spaces - i.e. approximately fifteen single space typewritten lines) (16)

ON 01-16-85 AT 2300 HOURS WITH THE REACTOR COOLANT SYSTEM IN MODE 5 (COLD SHUTDOWN) A SURVEILLANCE TEST WAS COMPLETED ON THE A TRAIN BATTERY WHICH REQUIRED THE A TRAIN BATTERY BE DECLARED INOPERABLE AS PER TECHNICAL SPECIFICATION 4.8.2.4.2. THE B TRAIN EMERGENCY DIESEL GENERATOR WAS ALSO INOPERABLE AT THIS TIME WHICH MADE THE B TRAIN BATTERY INOPERABLE IN THAT THE BASES OF 3.8.2.4 COULD NOT BE COMPLIED WITH WHICH REQUIRES THE SYSTEM BE CAPABLE OF OPERATING FOR AN EXTENDED PERIOD.

WHEN THE NONLICENSED MAINTENANCE SUPERVISOR COMMUNICATED THE IN-OPERABILITY OF THE A TRAIN BATTERY TO THE SENIOR REACTOR OPERATOR (SRO) IN CHARGE A MISUNDERSTANDING OF THE OPERABILITY OF THE A TRAIN BATTERY OCCURRED. THIS RESULTED IN THE A TRAIN BATTERY NOT BEING DECLARED INOPERABLE AND THE ACTION STATEMENT OF 3.8.2.4 NOT BEING COMPLIED WITHIN THE TIME REQUIRED.

THE ERROR WAS DISCOVERED AT 0710 ON 01-17-85 AND THE A TRAIN BATTERY MADE OPERABLE AT 0955 ON 01-17-85.

TO PREVENT THE RECURRENCE OF THIS MISUNDERSTANDING THE SURVEILLANCE PROCEDURES FOR THE BATTERIES WILL BE CHANGED. A REQUIREMENT WILL BE ADDED TO THE PROCEDURES THAT WILL FORMALIZE THE NOTIFICATION THAT THE BATTERIES DO NOT MEET THE MINIMUM REQUIREMENTS OF THE TECHNICAL SPECIFICATIONS. THE CHANGES TO THE PROCEDURES INVOLVED WILL BE COMPLETED BY THE END OF MARCH 1985.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  D. C. COOK NUCLEAR PLANT, UNIT 1	DOCKET NUMBER (2)  0 5   0 0   0 3   1 5	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 0   3	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 368A's) (17)

ON 01-16-85 AT 2300 HOURS WITH THE REACTOR COOLANT SYSTEM IN MODE 5 (COLD SHUTDOWN) A SURVEILLANCE TEST WAS COMPLETED ON THE A TRAIN BATTERY (BTRY). A SINGLE CELL (NUMBER 108) WAS FOUND TO HAVE A POTENTIAL DIFFERENCE BELOW THAT REQUIRED BY TECHNICAL SPECIFICATION 4.8.2.4.2. THIS SUBSTANDARD CELL MADE THE ENTIRE A TRAIN BATTERY INOPERABLE.

THE B TRAIN BATTERY WAS IN SERVICE AT THIS TIME AND ONLY ONE BATTERY TRAIN WAS REQUIRED IN MODE 5. HOWEVER AT THE SAME TIME THE A TRAIN BATTERY WAS INOPERABLE THE B TRAIN EMERGENCY DIESEL GENERATOR (DG) WAS INOPERABLE FOR MAINTENANCE. WITHOUT THE EMERGENCY DIESEL GENERATOR OPERABLE THE B TRAIN BATTERY WOULD NOT HAVE MAINTAINED THE D.C. VOLTAGE FOR AN EXTENDED PERIOD OF TIME AS IS STATED IN THE TECHNICAL SPECIFICATION BASES. WITHOUT THE EMERGENCY DIESEL GENERATOR THE B TRAIN BATTERY WAS INOPERABLE.

WITH BOTH TRAIN A AND TRAIN B BATTERIES INOPERABLE THE LIMITING CONDITION FOR OPERATION FOR TECHNICAL SPECIFICATION 3.8.2.4 REQUIRES THAT THE NECESSARY EQUIPMENT BE MADE OPERABLE OR CONTAINMENT INTEGRITY BE ESTABLISHED WITHIN EIGHT HOURS. THE TECHNICAL SPECIFICATION REQUIREMENT WAS COMPLIED WITH AT 0955 HOURS ON 01-17-85, TWO HOURS AND FIFTY FIVE MINUTES BEYOND THE REQUIRED EIGHT HOURS.

THE ACTION STATEMENT WAS NOT COMPLETED IN THE PRESCRIBED TIME LIMIT DUE TO A MISUNDERSTANDING BY THE SENIOR REACTOR OPERATOR (SRO) IN CHARGE OF THE OPERATING SHIFT AND THE NONLICENSED MAINTENANCE SUPERVISOR WHO REPORTED THE A TRAIN BATTERY PROBLEM. THE SRO UNDERSTOOD FROM THE CONVERSATION THAT THREE CELLS HAD TO FAIL BEFORE THE BATTERY WAS INOPERABLE WHICH WAS CONTRARY TO WHAT THE MAINTENANCE SUPERVISOR BELIEVED HE COMMUNICATED. THIS LEAD TO THE SRO NOT TAKING THE REQUIRED STEPS TO MEET THE ACTION STATEMENT.

AT 0710 HOURS ON 01-17-85 THE ERROR IN NOT DECLARING THE A TRAIN BATTERY INOPERABLE WAS DISCOVERED. CORRECTIVE ACTIONS WERE INITIATED AND THE A TRAIN BATTERY WAS RETURNED TO OPERABLE STATUS AT 0955 HOURS ON 01-17-85.

TO PREVENT THE RECURRENCE OF THIS MISUNDERSTANDING THE SURVEILLANCE PROCEDURES FOR THE BATTERIES WILL BE CHANGED. A REQUIREMENT WILL BE ADDED TO THE PROCEDURES THAT WILL FORMALIZE THE NOTIFICATION THAT THE BATTERIES DO NOT MEET THE MINIMUM REQUIREMENTS OF THE TECHNICAL SPECIFICATIONS. THE CHANGES TO THE PROCEDURES INVOLVED WILL BE COMPLETED BY THE END OF MARCH 1985.



**INDIANA & MICHIGAN ELECTRIC COMPANY**

DONALD C. COOK NUCLEAR PLANT  
P.O. Box 458, Bridgman, Michigan 49106  
(616) 465-5901

February 14, 1985

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

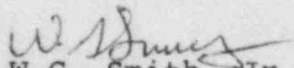
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Document Control Manager:

In accordance with the criteria established by 10CFR50.73  
entitled Licensee Event Reporting System, the following  
report/s are being submitted:

RO 85-003-0

Sincerely,

  
W.G. Smith, Jr.  
Plant Manager

/cbm

Attachment

cc: John E. Dolan  
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