

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Joseph M. Farley - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 4	PAGE (3) 1 OF 0 2
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TITLE (4)  
Refueling Water Level Less Than Required With Inoperable RHR Loop

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES
0	1	2 3 8 5	8 5	0 0 3	0 0	0 2	1 8 8	5	
									DOCKET NUMBER(S) 0 5 0 0 0

OPERATING MODE (9) 6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)								
POWER LEVEL (10) 0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)					
	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)					
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
	20.405(a)(1)(iii)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)						
	20.405(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)						
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)							TELEPHONE NUMBER		
NAME J. D. Woodard							AREA CODE		
							2 0 5 8 9 9 - 5 1 5 6		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO								

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On two occasions on 1-23-85, Technical Specification 3.0.4 was violated in that the water level above the reactor pressure vessel flange was decreased below twenty-three feet while thirteen snubbers had been removed for surveillance testing from the A train Residual Heat Removal (RHR) loop. Removal of the snubbers made the A train RHR loop inoperable from a Technical Specifications point of view even though it was capable of providing water to the reactor coolant system and the refueling canal. Technical Specifications require both trains of RHR to be operable in Mode 6 when the water level above the top of the reactor pressure vessel flange is less than twenty-three feet. Health/safety of the public was not affected.

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PDR ADOCK 05000364  
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Joseph M. Farley - Unit 2	DOCKET NUMBER (2)  0   5   0   0   0   3   6   4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0   0   3	- 0   0	0   2	OF	0   2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On two occasions on 1-23-85 Technical Specification 3.0.4 was violated in that the water level above the reactor pressure vessel flange was decreased below twenty-three feet while thirteen snubbers had been removed for surveillance testing from the A train Residual Heat Removal (RHR) loop. Removal of the snubbers made the A train RHR loop inoperable from a Technical Specifications point of view even though it was capable of providing water to the reactor coolant system and the refueling canal. Technical Specifications require both trains of RHR to be operable in Mode 6 when the water level above the top of the reactor pressure vessel flange is less than twenty-three feet.

Beginning at 0820 on 1-23-85, the water level in the refueling canal was reduced to a level approximately fifteen feet above the top of the reactor pressure vessel flange in preparation for FNP-2-STP-40 (Safety Injection With Loss of Offsite Power Test). The water level was increased above the Technical Specifications minimum value at approximately 1115, which was shortly after the initiation of FNP-2-STP-40. Beginning at 1304 on 1-23-85, the water level was decreased to the mid-plane of the reactor coolant loops and maintained at that point to allow the head to be placed on the reactor pressure vessel. The thirteen snubbers were replaced and the A train RHR was declared operable at 0102 on 1-25-85.

Operations personnel were aware that the snubbers had been removed from the A train RHR loop. The effect of the removed snubbers was discussed and it was incorrectly determined that the loop was able to perform its design function. The Operations personnel failed to recognize that reducing the water level below the minimum allowable value was a violation of Technical Specification 3.0.4.

This event was caused by personnel error. The personnel involved in this event have been counseled.

**Mailing Address**  
Alabama Power Company  
600 North 18th Street  
Post Office Box 2641  
Birmingham, Alabama 35291  
Telephone 205 783-6090

**R. P. McDonald**  
Senior Vice President  
Flintridge Building



Alabama Power  
*the southern electric system*

February 15, 1985

Docket No. 364

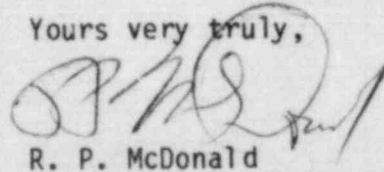
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Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 2, Licensee Event Report No. LER 85-003-00 is forwarded in accordance with 10CFR50.73 to provide 30 day written notification of the occurrence.

If you have any questions, please advise.

Yours very truly,



R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

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