

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Shoreham Nuclear Power Station Unit #1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 2	PAGE (3) 1 OF 3
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TITLE (4)  
Suspended Fire Watches

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																											
01	04	85	85	003	00	02	04	85	None		0 5 0 0 0																																											
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9)</td> <td style="width:15%;">5</td> <td colspan="10">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="6">POWER LEVEL (10) 0 0 0</td> <td></td> <td>20.402(b)</td> <td>20.406(c)</td> <td>50.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td></td> <td>20.406(a)(1)(i)</td> <td>50.36(e)(1)</td> <td>50.73(a)(2)(v)</td> <td>73.71(c)</td> </tr> <tr> <td></td> <td>20.406(a)(1)(ii)</td> <td>50.36(e)(2)</td> <td>50.73(a)(2)(vii)</td> <td><input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td></td> <td>20.406(a)(1)(iii)</td> <td>50.73(a)(2)(ii)</td> <td>50.73(a)(2)(viii)(A)</td> <td></td> </tr> <tr> <td></td> <td>20.406(a)(1)(iv)</td> <td>50.73(a)(2)(iii)</td> <td>50.73(a)(2)(viii)(B)</td> <td></td> </tr> <tr> <td></td> <td>20.406(a)(1)(v)</td> <td>50.73(a)(2)(iii)</td> <td>50.73(a)(2)(ix)</td> <td></td> </tr> </table>												OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)										POWER LEVEL (10) 0 0 0		20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)		20.406(a)(1)(i)	50.36(e)(1)	50.73(a)(2)(v)	73.71(c)		20.406(a)(1)(ii)	50.36(e)(2)	50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)		20.406(a)(1)(iii)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)			20.406(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)			20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME Joseph G. Wynne, Operational Compliance Engineer	TELEPHONE NUMBER AREA CODE: 5 1 6   9 2 9 - 8 3 0 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE: <input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH:    DAY:    YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

This report is pursuant to Special Report Requirement of Technical Specification 6.9.2.

On December 22, 1984 and seven other days following this date, hourly fire watch patrols were suspended for time periods longer than what was agreed to in SNRC 1122. These suspended fire patrols were due to an inconsistency, which was not recognized, between the approved startup procedure for fuel loading and the fire watch commitment set forth in SNRC 1122. When the discrepancy was discovered the startup procedure was changed.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Shoreham Nuclear Power Station Unit #1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 2	LER NUMBER (6)			PAGE (3)		
		YEAR 8 5	SEQUENTIAL NUMBER - 0 0 3	REVISION NUMBER - 0 0			
					0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

This report is pursuant to Special Report Requirements of Technical Specification 6.9.2.

During the time interval December 3-7, 1984, Shoreham Nuclear Power Station (SNPS) was given a special team inspection of fire protection, Inspection No. 50-322/84-46. Item 84-46-05 of the inspection describes a deviation regarding the number of detectors per area throughout the Reactor Building. In response to this inspection SNPS provided an immediate response to these items via SNRC letter 1122, dated December 7, 1984. The specific response to the issue of detector system design within the Reactor Building committed to providing hourly fire watch patrols in the areas protected by the detectors in accordance with the provisions of Technical Specification Action Statement 3.3.7.9.

Based on the SNRC Letter 1122, the commission issued a Confirmatory Action Letter CAL 84-25 dated December 7, 1985 stating that it was the NRC's understanding the compensatory measures described in the SNRC Letter 1122 would be fully implemented by December 8 and would remain in effect until permanent corrective actions approved by the NRC staff could be taken.

Upon receipt of the Low Power License on December 7, 1984, fire patrols were established within the Reactor Building. On December 22, 1984 the patrols were suspended on elevation 175 (the refueling floor) and the Primary Containment for two hours and twenty minutes. This action was done in accordance with an approved Startup procedure for conducting the fuel loading operation. The procedure had been written to remove people from these areas anytime control rod manipulations were ongoing as an added radiological safety precaution. The Watch Engineer decided personnel safety overrode requirement the for fire watches. The fire watches were also secured for the same reason on seven different days. On January 4, 1985 the discrepancy between the requirements to maintain the fire watches and the requirement to remove people from the areas was reconciled by plant management after detailed discussions. Subsequent to these discussions the decision was reviewed with two NRC inspectors. At this time the startup procedure was changed to permit fire watches to remain in the areas. These patrols were modified during control rod movements to ensure potential concerns were addressed.

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There appears to be no safety significance to this event for several reasons. First the detectors that are in place on the refueling floor and in the primary containment are operational, and would have detected a fire in the vicinity of the detectors. Secondly, on the refueling floor, although a roving fire watch was not in effect, there was a security guard continuously posted in the northeast corner of the floor to prevent access onto the floor. If a fire would have started, the security guard would have undoubtedly seen it and reported it. In the primary containment there are temperature elements with control room indicators which would have indicated any temperature rise to the Control Room operators. Finally as described in Supplements 5 and 6 to the Safety Evaluation Report for Phase I (Fuel Loading and Precriticality Testing) the NRC determined that there was no risk to the public health and safety because there can be no radiological consequences for all the events analyzed in Chapter 15 of the FSAR and these events bound any event which may have resulted for a fire in the areas in question.

In a separate but related event on January 23, fire watches were secured on elevation 175 of the Reactor Building for a period of one hour and twenty minutes and in the Screenwell Building for a period of one hour and thirty minutes during the declaration on an Unusual Event due to a bomb threat. The fire watches were removed from the areas while the security forces searched the areas for the bomb. Thus, even though the official fire watch was not in the area during this time security officers in search of a bomb were, and would have reported any detection of a fire. It is not felt that this occurrence constitutes any deviation from the commitments made in the SNRC letter 1122, and was prudent under the circumstances. The bomb threat is also discussed in LER 85-002.



# LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION • P.O. BOX 628 • WADING RIVER, NEW YORK 11792

TEL. (516) 929-8300  
PM-85-018

February 1, 1985

Dr. Thomas E. Murley  
Regional Administrator  
Region 1, U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Sir:

Pursuant to Technical Specification 6.9.2, this special report is hereby submitted with respect to securing fire watches committed to in SNRC letter 1122 dated December 7, 1984, and acknowledged by NRC confirmatory action letter 84-25 of the same date.

Very truly yours,

W. E. Steiger  
Plant Manager  
Shoreham Nuclear Power Station

WES/rwd

cc: Peter Eselgroth, Senior Resident Inspector  
Institute of Nuclear Power Operations  
American Nuclear Insurers

SR2-A43.700

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