

V-2-B-L  
Jerry Castro

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JAN 20 1974

Docket No. 50-219

Senator Clifford P. Case  
United States Senate

Dear Senator Case:

This is in further response to your communication of November 20, 1973 transmitting Mr. Kurtz's letter of November 15, 1973 regarding the operation of the Oyster Creek Nuclear Generating Station, and supplements our letter to you of December 10, 1973.

In his letter of November 15, 1973, Mr. Kurtz expressed concern for the effect of dredging spoils containing radioactivity on the health and safety of residents living near the dredged canal. These concerns arose from statements contained in a letter from Mr. John Russo, Chief, Bureau of Radiation Protection, New Jersey Department of Environmental Protection, to the ALC, dated August 22, 1973. Mr. Russo's letter stated that samples taken by the State from the sediment in the bottom of the discharge canal show a radioactivity level that could result in a "significant" dose to individuals on the canal banks if the canal were dredged and spoils were placed along the canal banks.

In subsequent discussions between the Commission and Mr. Russo, he pointed out that the New Jersey Department of Environmental Protection is not concerned at this time that this is a problem, but that his comment on the dredging spoils was made only to ensure that the AEC was made aware of a potentially sensitive situation. Mr. Russo stated that the State of New Jersey is, in fact, committed to monitor any dredging operations at Oyster Creek and to determine the final fate of any material taken from the canal in any future dredging operations.

In response to inquiries, the Jersey Central Power and Light Company has stated that no dredging has occurred in the discharge canal since before the Oyster Creek Station commenced operation. Mr. Russo also has said that the State knows of no dredging operations in the canal in recent years.

Data contained in a July 1973 report issued by the New Jersey Department of Environmental Protection indicate that the radioactivity in the Oyster Creek sediment is confined to the uppermost two inch layer.

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On the basis of the concentration found in the sediment and the conservative assumption that any dredging operation would effectively mix the radionuclide-rich layer with ten times its volume of non-active sediment, it is estimated that a dose of up to 10 millirems per year could be received by an individual who resided continuously on the spoils pile. This is a small fraction of the average dose to an individual from natural background radiation of 125 millirems per year. However, the upper range of this dose estimate could exceed the Commission's as-low-as-practicable guidelines for radiation exposure. Accordingly, as indicated above, any dredging operations will be carried out under close surveillance, and disposition of spoils will be made in a manner and location which minimizes exposure to the general public.

The Commission, through periodic radiological monitoring which has been required by the technical specifications in the Oyster Creek Operating License since operation began, will continue to maintain surveillance over the radioactivity levels in the canal as well as all other aspects of plant operation relating to the health and safety of the public. The Commission requires that any operations at the plant which could potentially result in radiation exposure to the public will be conducted in such a way as to keep these exposures within existing limits and as low as practicable.

Sincerely,

Original Signed By

A. Giambusso

A. Giambusso, Deputy Director  
for Reactor Projects  
Directorate of Licensing

SEE ATTACHED YELLOW FOR PREVIOUS CONCURRENCE


OFFICE	L:EP-4	ICEP-4	OGC	L:EP	L:RP	OCR
SURNAMES	RBBeynon <i>RBB</i>	WRegan <i>WR</i>		DR <i>DR</i>	AGiambusso	
DATE	2/15/74	2/15/74	2/ /74	2/ /74	2/ /74	2/ /74

V-2E1

# Jersey Central Power & Light Company

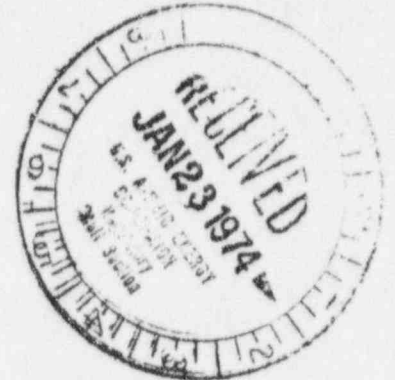


MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111

MEMBER OF THE  
 General  Public Utilities Corporation

January 18, 1974

Mr. Dennis L. Ziemann, Chief  
 Operating Reactors Branch #2  
 Directorate of Licensing  
 United States Atomic Energy Commission  
 Washington, D. C. 20545



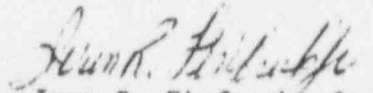
Dear Mr. Ziemann:

This is in response to your letter of December 19, 1973 requesting electromatic relief valve operational data regarding our Oyster Creek Nuclear Generating Station. Attached herewith is a summary tabulation of electromatic relief valve operation covering the period of plant startup testing and operation from January 1, 1969 to present.

It is understood that this letter is in response to Item 1 of your request and we intend to respond to Item 2 before the specified deadline.

It is also our intention to report future relief valve operation in our facility semi-annual operating reports in a similar manner.

Very truly yours,

  
 Ivan R. Finfrock, Jr.  
 Vice President

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Attachment

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Summary of Electromatic Relief Valve Operation  
January 1, 1969 to December 31, 1969

Date of Operation: June 30, 1969

Purpose of Operation: Operability Test

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation: The reactor was subcritical.

Comments: Reactor trip: low water level, drywell high pressure, and core spray booster pump discharge pressure signals were created to cause automatic operation of the relief valves.

Date of Operation: August 11, 1969

Purpose of Operation: Operability Test

Mode of Initiation: Automatic

Reactor Condition Prior to Operation: The reactor was subcritical.

Comments: Reactor triple low water level, drywell high pressure, and core spray booster pump discharge pressure signals were created to cause automatic operation of the relief valves.

Date of Operation: September 21, 1969

Purpose of Operation: Startup Test #16

Mode of Initiation: Manual

Reactor Conditions Prior to Operation:

Steam Flow -  $1.2 \times 10^6$  lbm/hr  
Reactor Pressure - 980 psig  
Electrical Output - 385 MW(e)

Comments: Each of the four (4) electromatic relief valves were independently tested and found to discharge  $.5 \times 10^6$  lbm/hr (8.55% rated steam flow) at the above conditions.

Summary of Electromatic Relief Valve Operation  
January 1, 1970 to December 31, 1970

Date of Operation: February 15, 1970

Purpose of Operation: Primary coolant system depressurization.

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation:

Steam Flow -  $4.95 \times 10^6$  lbm/hr  
Reactor Pressure - 985 psig  
Electrical Output - 450 MW(e)

Comments: The reactor scrambled due to a high neutron flux signal following a turbine trip. The steam pressure subsequently increased to the set point of the electromatic relief valves (1125 psig) causing them to actuate.

Date of Operation: December 25, 1970

Purpose of Operation: Primary coolant system depressurization.

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation:

Steam Flow -  $6.12 \times 10^6$  lbm/hr  
Reactor Pressure - 1000 psig  
Electrical Output - 582 MW(e)

Comments: The reactor underwent an anticipatory scram due to a momentary closure of all four (4) main stop valves. The steam pressure subsequently increased to the set point of the electromatic relief valves (1125 psig) causing them to actuate.

Summary of Electromatic Relief Valve Operation  
January 1, 1971 to December 31, 1971

Date of Operation: November 7, 1971

Purpose of Operation: Operability Test

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation: The reactor was subcritical.

Comments: Reactor triple low water level, drywell high pressure, and core spray booster pump discharge pressure signals were created to cause automatic operation of the relief valves.

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Summary of Electromatic Relief Valve Operation  
January 1, 1972 to December 31, 1972

Date of Operation: December 29, 1972

Purpose of Operation: Primary coolant system depressurization.

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation:

Steam Flow -  $7.35 \times 10^6$  lbm/hr  
Reactor Pressure - 1000 psig  
Electrical Output - 645 MW(e)

Comments: The reactor scrambled as the result of a generator trip from a loss of field. The reactor pressure increased to 1070 psig, which opened the electromatic relief valves, one of which did not reseal and the vessel depressurized to 200 psig in 60 minutes.

Summary of Electromatic Relief Valve Operation  
January 1, 1973 to December 31, 1973

Date of Operation: May 28, 1973

Purpose of Operation: Operability Test

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation: The reactor was subcritical.

Comments: Reactor triple low water level, drywell high pressure, and core spray booster pump discharge pressure signals were created to cause automatic operation of the relief valves.

Date of Operation: June 30, 1973

Purpose of Operation: Primary coolant system depressurization.

Mode of Initiation: Automatic

Reactor Conditions Prior to Operation:

Steam Flow -  $7.45 \times 10^6$  lbm/hr  
Reactor Pressure - 1020 psig  
Electrical Output - 625 MW(e)

Comments: The reactor scrambled due to a generator trip. The reactor pressure subsequently increased to the set point of the electromatic relief valves of 1070 psig, causing them to open.