

50-219-156

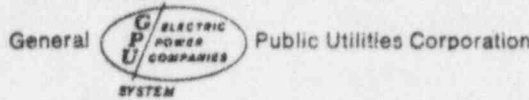
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MEMBER OF THE



January 25, 1974



Mr. A. Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
United States Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. Giambusso:

Subject: Oyster Creek Station
Docket No. 50-219
Abnormal Occurrence Report No. 50-219/74/5

The purpose of this letter is to forward to you the attached Abnormal Occurrence Report in compliance with paragraph 6.6.2.a of the Technical Specifications.

Enclosed are forty copies of this submittal.

Very truly yours,

Donald A. Ross
Manager, Nuclear Generating Stations

cs
Enclosures

cc: Mr. J. P. O'Reilly, Director
Directorate of Regulatory Operations, Region I

B/655

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/5

Report Date:

January 25, 1974

Occurrence Date:

January 16, 1974

Identification of Occurrence:

Violation of the Technical Specifications, paragraph 4.5.F.1.d, failure of Main Steam Isolation Valve NS04A to meet the allowable leakage requirements. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraphs 1.15D & E.

Conditions Prior to Occurrence:

The plant was shut down with the reactor coolant less than 212°F and the reactor mode switch in "Refuel".

Description of Occurrence:

It was necessary to operate NS04A in order to provide adequate ventilation of the reactor vessel while performing maintenance work on Electromatic Relief Valve NR-108-D after the plant shutdown. Valve NS04A, therefore, was not tested in the "as found" condition after initial closure.

1447 - Leak rate test on Main Steam Isolation Valve NS04A began.

1517 - Leak rate test on Main Steam Isolation Valve NS04A ended.

Leakage rate was 31.7 SCFH, corrected to 20 psi. The maximum allowable leakage rate is 9.945 SCFH, as required by Technical Specifications, paragraph 4.5.F.1.d.

Apparent Cause of Occurrence:

The cause of this occurrence is attributed to component failure. After checking the test assembly and the components of the MSIV, it was determined that the packing around the valve shaft was the cause of the excessive leak rate.

Analysis of Occurrence:

The safety significance of the failure of NS04A to pass the leakage rate test was a loss of redundancy in an engineered safety feature designed to minimize the release of fission products under design bases accident conditions.

Corrective Action:

The valve shaft was repacked and the valve was retested successfully. NS03A was also retested to insure that it had an acceptable leak rate, since its test assumes that the valve NS04A has negligible leakage. The retest of NS03A indicated no detectable leakage.

MSIV NS04A has been selected to undergo a complete preventative maintenance inspection during the refueling outage presently scheduled for April 1974.

Failure Data:

The valve stem packing on NS04A failed on September 27, 1973 (letter to Mr. A. Giambusso from Mr. D. A. Ross, dated October 12, 1973). At that time, the valve was repacked and subsequently passed its leak rate test.