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ARTHUR E. LUNDVALL, JR.
VICE PRESIDENT
SUPPLY

February 22, 1985

U. S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D.C. 20555

ATTENTION: Mr. James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

SUBJECT: Calvert Cliffs Nuclear Power Plant Unit 1; Docket No. 50-317
Amendment to Operating License DPR-53
Eighth Cycle License Application

REFERENCE: A. E. Lundvall, Jr., to J. R. Miller letter dated December 31, 1984, Clad
Collapse/Augmentation Factor Analysis

ENCLOSURE: Unit 1 Cycle 8 Reload License Submittal

Gentlemen:

The Baltimore Gas & Electric Company requests an amendment to its Operating License, No. DPR-53 for Calvert Cliffs Unit 1 to allow operation for an eighth cycle. The enclosed report presents a detailed description of the required Standard Technical Specifications (STS) with supporting safety analysis information to ensure conservative operation at a rated thermal power of 2700 MWth for Unit 1 Cycle 8.

In the reference we requested a change to the augmentation factor Technical Specifications for Unit 1 and Unit 2, and a PDIL change for Technical Specification change for Unit 1. Subsequent analysis has shown that the PDIL Technical Specification is no longer required and we withdraw the request to change the PDIL limits. Additionally, the Technical Specification change requested for Unit 2 augmentation factors will not become applicable until the Unit 2 shutdown at the end of Cycle 6 in October of 1985. Consequently, we request that the application of the augmentation factors to the Unit 2 Technical Specification be delayed until the end of Unit 2 Cycle 6.

Our present intention is to begin the Unit 1 refueling outage on April 5, 1985, and to complete the outage and begin the approach to criticality on May 22, 1985, with return to power operations immediately thereafter.

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w/ check #150⁰⁰
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PROPOSED CHANGES (BG&E FCR 85-3001)

The majority of the reload is enveloped by the analyses for the reference cycle, Unit 1 Cycle 7. The major changes are: STS 3.1.1.1, decreases the required shutdown margin; STS 3.1.1.4, changes the allowed moderator temperature coefficients; STS 4.2.1.4, removes the flux peaking augmentation factors; and STS 4.5.2.h, revises the HPSI flow balance requirements. The first two changes are supported by a reanalysis of seven design basis events (enclosure section 7) and the CEA reconfiguration. The third Technical Specification change is explained in the reference.

The change to the HPSI flow valve requirement results in a slight reduction to the assumed high pressure safety injection flow. The lower flow is compensated by crediting partial injection flow from one charging pump and by revising the axial power distribution Technical Specification. These changes were all incorporated in the reanalysis of the Small Break Loss of Coolant Accident presented in section 8 of the enclosure.

DETERMINATION OF SIGNIFICANT HAZARDS

We have determined, based on the analytical information supplied in the enclosure, that this amendment request does not involve a significant hazards consideration. The proposed changes are consistent with example (iii) of amendments considered not likely to involve significant hazards consideration, as shown in the Federal Register notice dated April 6, 1983, page 14870. No fuel assemblies to be loaded into the Cycle 8 core will be of new or different design than those used previously and found to be acceptable to the NRC. No proposed changes to the Technical Specifications for Cycle 8 involve acceptance criteria which are significantly different from those previously found acceptable to the NRC. The analytical methods used to determine conformance with the Technical Specifications and regulations are consistent with previous NRC approvals and involve no significant changes.

We conclude that the proposed reload license amendment does not involve a significant hazard consideration in that:

1. The probability or consequences of an accident evaluated is not significantly increased. Six non-LOCA design basis events were re-analyzed. Their consequences were not significantly increased and all were within acceptance criteria. Both the large break loss of coolant accident and the small break loss of coolant accident were re-analyzed and their consequences were found to be slightly improved.
2. The reload application does not create the possibility of a new or different kind of accident from any previously evaluated.
3. The license reload does not involve a significant reduction in the margin for safety. Small changes in shutdown margin and increase in moderator temperature coefficients are compensated by the CEA reconfiguration and the reanalysis performed in the enclosure.

Mr. James R. Miller
February 22, 1985
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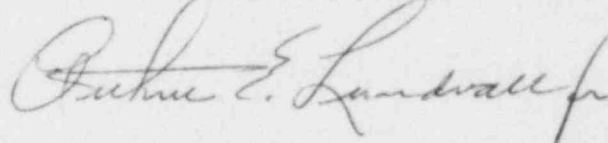
SAFETY COMMITTEE REVIEW

These proposed changes to the Technical Specifications and the determination of the significant hazards have been reviewed by our Plant Operations and Off-Site Safety Review Committees, and they have concluded that the implementation of these changes will not result in an undue risk to the health and safety of the public.

FEE DETERMINATION

Pursuant to 10CFR 170.21, we are including BG&E check number A303000 in the amount of \$150.00 to cover the application fee for this request.

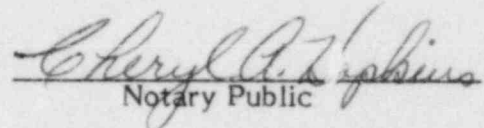
Very truly yours,



STATE OF MARYLAND:
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CITY OF BALTIMORE : **TO WIT:**

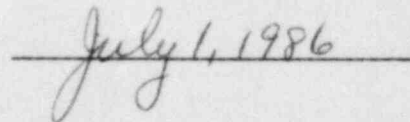
Arthur E. Lundvall, Jr., being duly sworn states that he is Vice President of the Baltimore Gas and Electric Company, a corporation of the State of Maryland; that he provides the foregoing response for the purposes therein set forth; and the statements made are true and correct to the best of his knowledge, information, and belief; and that he was authorized to provide the response on behalf of said Corporation.

WITNESS my Hand and Notarial Seal:



Notary Public

My Commission Expires:



AEL/JAM/lmt

Enclosures

cc: D. A. Brune, Esq.
G. F. Trowbridge, Esq.
D. H. Jaffe, NRC
T. Foley, NRC