



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 14, 1996

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations (SA2A)
Florida Power Corporation
ATTN: Manager, Nuclear
Licensing
15760 W Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER, UNIT 3 - EVALUATION OF THE SECOND 10-YEAR INTERVAL
INSPECTION PROGRAM PLAN REQUESTS FOR RELIEF NOS. 95-020, 95-030, AND
95-040 (TAC NO. M93755)

Dear Mr. Beard:

By letter dated September 22, 1995, you requested approval of Requests for Relief (RR) Nos. 95-010, 95-020, 95-030, and 95-040 related to the second 10-year interval inservice inspection program plan for Crystal River Nuclear Plant, Unit 3. Additional information was provided by the licensee in its letters dated February 16, 1996, and March 12, 1996.

We have completed our review of RR Nos. 95-020, 95-030, and 95-040, with technical assistance from our contractor, the Idaho National Engineering Laboratory (contractor). The staff's evaluation and conclusions are contained in Enclosure 1. We concur and adopt the recommendations of our contractor's Technical Letter Report, which is Enclosure 2.

The licensee's alternative contained in RR No. 95-020 is authorized pursuant to 10 CFR 50.55a(a)(3)(ii), provided that all requirements of Code Case N-498-1 are satisfied. This relief is authorized until such time as the Code Case is published in a future revision of Regulatory Guide 1.147. At that time, if the licensee intends to continue to implement this Code Case, the licensee is required to follow all provisions in Code Case N-498-1, with limitations issued in Regulatory Guide 1.147, if any.

For RR No. 95-030, the staff concludes that requiring the licensee to perform the Code-required examination of the reactor pressure vessel transition piece-to-bottom head weld with the examination device currently being deployed for in-vessel examinations, will result in a burden without a compensating increase in quality and safety, because there is a potential for damage to occur if the examination device impacts the in-vessel components. Therefore, the licensee's proposed alternative contained in RR No. 95-030 is authorized pursuant to 10 CFR 50.55a(a)(3)(ii) for the second interval only, on a one-time basis.

The licensee request to use the 1989 Edition, Section XI, Appendix I, Supplement 3 regarding examination of Class 1 and 2 ferritic piping with diameters greater than 20 inches is authorized in accordance with

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Mr. Percy M. Beard, Jr.

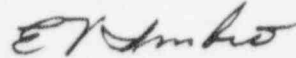
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10 CFR 50.55a(g)(4)(iv) provided all of the provisions of Appendix I, Article I-2000, Supplement 3, are met.

Our review of RR No. 95-010 is in progress and will be evaluated in a separate document.

If you have any questions please call at (301) 415-1471.

Sincerely,



Eugene V. Imbro, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures:

1. Safety Evaluation
2. Technical Letter Report

cc w/enclosures:

See next page

Mr. Percy M. Beard, Jr.

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(Original Signed By)
Eugene V. Imbro, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-302

Distribution

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- 1. Safety Evaluation
- 2. Technical Letter Report

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cc w/enclosures:
See next page

*See previous concurrence
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Florida Power Corporation

Crystal River Unit No. 3
Generating Plant

cc:

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