# OPERATING DATA REPORT

DOCKET NO. 50-334
DATE 2/2/85
COMPLETED BY R. A. Smith
TELEPHONE 412-643-1825

## OPERATING STATUS

1. Unit Name: Beaver Valley Power Station, Unit #1
2. Reporting Period:
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 923
5. Design Electrical Rating (Net MWe): 835
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

| | This Month | Yr.-to-Date | Cumulative |
|---|---|---|---|
| 11. Hours In Reporting Period | 744 | 744 | 76,752 |
| 12. Number Of Hours Reactor Was Critical | 684.1 | 684.1 | 38,039.8 |
| 13. Reactor Reserve Shutdown Hours | 0 | 0 | 4,482.8 |
| 14. Hours Generator On-Line | 616.9 | 616.9 | 36,699.8 |
| 15. Unit Reserve Shutdown Hours | 0 | 0 | 0 |
| 16. Gross Thermal Energy Generated (MWH) | 1,230,666 | 1,230,666 | 84,629,180 |
| 17. Gross Electrical Energy Generated (MWH) | 396,000 | 396,000 | 26,890,400 |
| 18. Net Electrical Energy Generated (MWH) | 363,040 | 363,040 | 24,987,883 |
| 19. Unit Service Factor | 82.9 | 82.9 | 50.0 |
| 20. Unit Availability Factor | 82.9 | 82.9 | 50.0 |
| 21. Unit Capacity Factor (Using MDC Net) | 60.2 | 60.2 | 43.6 |
| 22. Unit Capacity Factor (Using DER Net) | 58.4 | 58.4 | 42.3 |
| 23. Unit Forced Outage Rate | 5.7 | 5.7 | 26.8 |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
N/A

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

| | Forecast | Achieved |
|---|---|---|
| INITIAL CRITICALITY | N/A | N/A |
| INITIAL ELECTRICITY | N/A | N/A |
| COMMERCIAL OPERATION | N/A | N/A |



IE24
1/1
(9/77)

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-334
UNIT BVPS Unit #1
DATE 2/8/85
COMPLETED BY P. A. Smith
TELEPHONE (412) 643-1825

MONTH January

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1 | 0 | 17 | 416 |
| 2 | 0 | 18 | 743 |
| 3 | 0 | 19 | 455 |
| 4 | 7 | 20 | 701 |
| 5 | 170 | 21 | 782 |
| 6 | 375 | 22 | 783 |
| 7 | 373 | 23 | 826 |
| 8 | 333 | 24 | 93 |
| 9 | 293 | 25 | 413 |
| 10 | 413 | 26 | 413 |
| 11 | 453 | 27 | 699 |
| 12 | 497 | 28 | 825 |
| 13 | 620 | 29 | 783 |
| 14 | 784 | 30 | 825 |
| 15 | 824 | 31 | 825 |
| 16 | 498 | | |

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-334
UNIT NAME BVPS Unit #1
DATE
COMPLETED BY P. A. Smith
TELEPHONE (412) 643-1825

REPORT MONTH January 1985

| No. | Date | Type[1] | Duration (Hours) | Reason[2] | Method of Shutting Down Reactor[3] | Licensee Event Report # | System Code[4] | Component Code[5] | Cause & Corrective Action to Prevent Recurrence |
|---|---|---|---|---|---|---|---|---|---|
| 1. | 850101 | S | 85.5 | C | 4 | N/A | ZZ | ZZZZZZZ | Station remained shutdown for 4th refueling. |
| 2. | 850104 | S | 4.5 | B | 1 | N/A | HA | TURBIN | At 0002 hours on the 4th, the Station was taken off line to perform the scheduled Turbine Overspeed Trip Test. The Station returned to service at 0432 hours on the 5th. |
| 3. | 850116 | F | 17.5 | A | 3 | 85-003 | CH | CKTBRK | A Reactor Trip with Safety injection occurred at 1525 hours. This was a result of Power Feed Breaker 3-3 from the vital bus to a portion of protection Channel 3 opening. |
| 4. | 850118 | S | 0 | B | 5 | N/A | CH | PUMPXX | A power reduction occurred at 2000 hrs. on the 18th for seal replacement on the Main Feedwater Pump [FW-P-1B]. The pump was returned to service at 0617 hours on the 20th, and power was subsequently increased. |

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continued From Previous Month
5-Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5
Exhibit I - Same Source

(9/77)

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-334
UNIT NAME BVPS Unit #1
DATE
COMPLETED BY P. A. Smith
TELEPHONE (412) 643-1825

REPORT MONTH January 1985

| No. | Date | Type[1] | Duration (Hours) | Reason[2] | Method of Shutting Down Reactor[3] | Licensee Event Report # | System Code[4] | Component Code[5] | Cause & Corrective Action to Prevent Recurrence |
|---|---|---|---|---|---|---|---|---|---|
| 5. | 850124 | F | 19.6 | A | 1 | 85-002 | CB | VALVEX | At 0239 hours on the 24th commenced plant shutdown due to loss of seal water to Reactor Coolant Pump [RC-P-1A]. The packing follower on valve CH-184 broke allowing packing to blow-out.<br><br>The valve CH-184 was repaired and the Station returned to service at 2310 hours on the 24th. |
| 6. | 850125 | F | 0 | B | 9 | N/A | CH | PUMPXX | Power was held at a reduced level so that alignment work could be performed on the 1B Main Feedwater Pump. Work was completed at 2325 hours on the 26th. |

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
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Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5
Exhibit I - Same Source

(9/77)

Duquesne Light

Nuclear Group
P.O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

February 8, 1985

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
Monthly Operating Report

U. S. Nuclear Regulatory Commission
Director, Office of Management
Information & Program Control
Washington, D. C. 20555

Gentlemen:

In accordance with Appendix A, Technical Specifications, the Monthly Operating Report is submitted for the month of January, 1985.

Very truly yours,

J. J. Carey
Vice President
Nuclear Group

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Enclosures

cc: NRC Regional Office
King of Prussia, PA

IE24
1/1

NARRATIVE SUMMARY OF

MONTHLY OPERATING EXPERIENCE

January 1985

January 1 — The Station was in Operation Mode 3, hot standby. The Reactor Coolant System temperature and pressure were being increased to normal operating level.

January 2 through January 3 — Normal operating pressure, 2235 psig, and temperature, 547°F were reached and maintained. At 0202 hours on the 2nd, criticality was achieved with Control Bank "D" at 158 steps. Core Low Power Physics testing was in progress.

January 4 — At 0529 hours the turbine was latched. The turbine was tripped at 0817 due to smoke issuing from an area between the #2 Low Pressure Turbine and the Generator. The smoke occurred due to high temperatures in the seal oil system. Turbine startup was reintiated at 0920 hours. At 0925 hours, the turbine was tripped due to an electrohydraulic system leak. Turbine startup recommenced at 1200 hours and the turbine trip test performed at 1231 hours. During the test an extraction steam non-return valve did not operate properly and retest was scheduled. At 1330 hours, the Main Unit Generator was synchronized to the grid. Power level was increased to 20 percent and maintained there for additional testing to be performed.

January 5 — At 0002 hours on the 5th, the Main Unit Generator was taken off line to perform the Turbine Overspeed Trip Test. The test completed satisfactorily at 0048 hours and the generator sychronized to the grid at 0432 hours. Reactor Power was increased at a 3 percent/hour rate.

January 6 — Reactor Power was at a 49 percent hold point to perform a full-core flux map.

January 7 — The Reactor Coolant System was at normal operating pressure and temperature at 49 percent power. Operations determined Rod F-10 indicator was reading low by greater than ± 12 steps from CBD Group Demand Counters. Verification using primary voltage also indicated Rod F-10 was low by greater than ± 12 steps from CBD Group Demand Counters. Rod overstepping procedure TOP 85-1 was performed to confirm that Rod F-10 was in alignment with the remaining rods in its respective group. Preparation was made to recalibrate the indicator for Rod F-10; however problems with the input test signal equiment were encountered.

January 8 through January 11 — Equipment to input test signal repaired and Rod F-10 was recalibrated to indicate proper position. Reactor Power was reduced to 30 percent due high steam generator conductivity. Individual condenser waterboxes were drained and tubes repaired, as necessary. Power level was then increased to 60 percent.

Reactor power was being maintained at 60 percent to perform flux maps for calibration of Power Range Detectors N-41, 42, 43 and 44.

January 12 through January 14 — Power level was increased at 3 percent/hour rate. At 0530 hours on the 14th power level of 100 percent was attained.

January 15 through January 16 — The Station was in Operational Mode 1 at a nominal 100 percent, and the Reactor Coolant System was at normal operating temperature and pressure. At 1528 hours on the 16th, a rapid drop of steam generator level and steam pressure in conjunction with a steam flow-feed mismatch caused a reactor trip with safety injection. The problem was attributed to the power feed breaker from vital bus 3 opening.

January 17 — Station in Operational Mode 3. At 0707 hours the reactor was taken critical. At 0902 hours the Main Unit Generator was synched to the grid. At 2230 hours the station reached 100 percent nominal power.

January 18 through January 19 — The Station was in Operational Mode 1 at a nominal 100 percent power. Power level was reduced to 60 percent at 2115 hours for performance of maintenance on Main Feedwater Pump [FW-P-1B].

January 20 through January 23 — The Station was in Operational Mode 1 with Reactor Power a nominal 100 percent. The Reactor Coolant System was at normal operating temperature and pressure.

January 24 — At 0239 hours commenced plant shutdown due to loss of seal water to Reactor Coolant Pump [RC-P-1A]. The Station was taken off line at 0331 hours and entered Mode 3 at 0410 hours. The Reactor was taken critical at 2141 hours and the Main Unit Generator was synched to the grid at 2310.

January 25 through January 26 — Station in Operational Mode 1 and at 55 percent power level. Maintenance being performed on Main Feedwater Pump [FW-P-1B].

January 1985
Summary of Monthly Operating Experience

| | |
|---|---|
| January 27 through January 31 | The Station was in Operational Mode 1 with Reactor Power at a nominal 100 percent. Reactor Coolant System was at normal operating pressure and temperature. |

MAJOR MAINTENANCE - January 1985

1. Maintenance of Main Feedwater Pump [FW-P-1B]