UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated December 14, 1984, as supplemented January 31, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 14, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-19
3/4 3-36
3/4 3-39
3/4 4-4a

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEMS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES IN WHICH SURVEILLANCE REQUIRED
3. REACTOR BUILDING SPRAY				
a. Reactor Building Pressure High-High coincident with HPI Signal	S	R	M(4)	1, 2, 3
b. Automatic Actuation Logic	N/A	N/A	M(1)(3)(5)	1, 2, 3
4. OTHER SAFETY SYSTEMS				
a. Reactor Building Purge Exhaust Duct Isolation on High Radioactivity				
1. Gaseous	S	Q	M	All Modes
b. Steam Line Rupture Matrix				
1. Low SG Pressure	N/A	R#	N/A	1, 2, 3
2. Automatic Actuation Logic	N/A	N/A	M(3)	1, 2, 3
c. Emergency Feedwater				
1. MFW Pump Turbine A and B Control Oil Low	S	R	N/A	1, 2, 3
2. OTSG A and B Level Low-Low	S	R	N/A	1, 2, 3,4

#The specified 18 month calibration frequency may be waived for Cycle V provided the surveillance is performed during Refuel V.

TABLE 4.3-2 (Cont'd)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES IN WHICH SURVEILLANCE REQUIRED	
5. REACTOR BUILDING ISOLATION					
a. Manual Initiation	N/A	N/A	R	5 or 6	|
b. Reactor Building Pressure High	S	R	M(2)	1, 2, 3	
c. Automatic Actuation Logic	N/A	N/A	M(1)(3)(5)	1, 2, 3, 4	|
d. Manual Initiation (HPI Isolation)	N/A	N/A	R	5 or 6	|
e. RCS Pressure Low (HPI Isolation)	S	R	M	1, 2, 3	
f. Automatic Actuation Logic (HPI Isolation)	N/A	N/A	M(1)(3)(5)	1, 2, 3, 4	|

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TABLE 3.3-9

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

INSTRUMENT	READOUT LOCATION	MEASUREMENT RANGE	MINIMUM CHANNELS OPERABLE
1. Reactor Trip Breaker Indication	CRD switch gear room 124 foot elevation	open-close	1 per trip breaker and 1 per secondary trip breaker
2. Reactor Coolant Temperature - Th	4160ES-B switchgear room 108 foot elevation	520-620°F	1 per loop
3. Reactor Coolant Pressure	4160ES-B switchgear room 108 foot elevation	0-2500 psig	1
4. Pressurizer Level	4160ES-B switchgear room 108 Foot elevation	0-320" H_2O	1
5. Steam Generator Pressure	4160ES-B switchgear room 108 foot elevation	0-1200 psig	1 per steam generator
6. Steam Generator Level	4160ES-B switchgear room 108 foot elevation	0-250" H_2O	1 per steam generator
7. Decay Heat Removal Temperature	4160ES-B switchgear room 108 foot elevation	0-300°F	1 per cooler
8. Motor driven Emergency Feedwater Pressure	Intermediate Building 95 foot elevation	0-2000 psig	1 per pump
9. Nuclear Services Closed Cycle Cooling Pumps Discharge Pressure	Auxiliary Building 95 foot elevation	0-300 psig	1
10. Nuclear Services Closed Cycle Cooling Cooler Outlet Temperature	Auxiliary Building 95 foot elevation	0-250°F	1 per cooler

TABLE 4.3-6

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1.	Reactor Trip Breaker Indication	M	N.A.
2.	Reactor Coolant Temperature-Th	M	R
3.	Reactor Coolant Pressure	M	R
4.	Pressurizer Level	M	R#
5.	Steam Generator Level	M	R
6.	Steam Generator Pressure	M	R#
7.	Decay Heat Removal Temperature	M	R
8.	Motor Driven Emergency Feedwater Pressure	M	R
9.	Nuclear Services Closed Cycle Cooling Pumps Discharge Pressure	M	R
10.	Nuclear Services Closed Cycle Cooling Cooler Outlet Temperature	M	R

#The specified 18 month calibration frequency may be waived for Cycle V provided the surveillance is performed during Refuel V.

TABLE 4.3-7

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1.	Power Range Nuclear Flux	M	Q*
2.	Reactor Building Pressure	M	R
3.	Source Range Nuclear Flux	M	R*
4.	Reactor Coolant Outlet Temperature	M	R
5.	Reactor Coolant Total Flow Rate	M	R
6.	RC Loop Pressure	M	R
7.	Pressurizer Level	M	R#
8.	Steam Generator Outlet Pressure	M	R#
9.	Steam Generator Level (Primary EFW Flow Detector)	M	R
10.	Borated Water Storage Tank Level	M	R
11.	Startup Feedwater Flow Rate	M	R#
12.	Reactor Coolant System Subcooling Margin Monitor	M	R
13.	PORV Position Indicator (Primary Detector)	M	R
14.	PORV Position Indicator (Backup Detector)	M	R
15.	PORV Block Valve Position Indicator	M	R
16.	Safety Valve Position Indicator (Primary Detector)	M	R
17.	Safety Valve Position Indicator (Backup Detector)	M	R
18.	Emergency Feedwater Ultrasonic Flow Indicator (Backup EFW Flow Detector)	M	R

*Neutron detectors may be excluded from CHANNEL CALIBRATION

#The specified 18 month calibration frequency may be waived for Cycle V provided the surveillance is performed during Refuel V.

REACTOR COOLANT SYSTEM

POWER OPERATED RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.3.2 The power operated relief valve (PORV) and its associated block valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

a. With the PORV inoperable, within 1 hour either restore the PORV to OPERABLE status or close the associated block valve and remove power from the block valve; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With the block valve inoperable, within 1 hour either restore the block valve to OPERABLE status or close the block valve and remove power from the block valve or close the PORV and remove power from the associated solenoid valve; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.3.2.1 In addition to the requirements of Specifications 4.0.5, the PORV shall be demonstrated OPERABLE at least once per 18 months by performance of a CHANNEL CALIBRATION.#

4.4.3.2.2 The block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel.

#The specified 18 month calibration frequency may be waived for Cycle V provided the surveillance is performed during Refuel V.