

U.S. ATOMIC ENERGY COMMISSION  
DIRECTORATE OF REGULATORY OPERATIONS  
REGION I

RO Inspection Report No: 50-219/74-10 Docket No: 50-219  
Licensee: Jersey Central Power and Light Company License No: DPR-16  
Madison Avenue and Punch Bowl Road Priority: \_\_\_\_\_  
Morristown, New Jersey 07960 Category: C  
Location: Oyster Creek, Forked River, New Jersey

Type of Licensee: 1930 MWt, BWR (GE)  
Type of Inspection: Announced  
Dates of Inspection: May 30-31, 1974  
Dates of Previous Inspection: May 8-9, 1974

Reporting Inspector: *T. Rebelowski*  
T. Rebelowski, Reactor Inspector

6/12/74  
Date

Accompanying Inspectors: \_\_\_\_\_  
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Date  
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Date  
\_\_\_\_\_  
Date  
\_\_\_\_\_  
Date

Other Accompanying Personnel: \_\_\_\_\_

\_\_\_\_\_  
Date

Reviewed By: *E. C. McCabe, Jr.*  
E. C. McCabe, Senior Reactor Inspector, Nuclear  
Support Section, Reactor Operations Branch

6/12/74  
Date

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## SUMMARY OF FINDINGS

### Enforcement Action

None

### Licensee Action on Previously Identified Enforcement Items

Not inspected

### Unusual Occurrences

The licensee identified reactor coolant leakage existing in the area of In Core Flux Housing, core position 28-05. (Details, Paragraph 4)

### Other Significant Findings

#### A. Current

The licensee postponed the Containment Integrated Leak Rate test due to the identification of in-core flux monitor housing leak. (Details, Paragraph 4)

#### B. Status of Previously Reported Unresolved Items

The licensee has revised the Integrated Primary Containment Leak Rate Test to resolve inspector concerns. (Details, Paragraph 3)

### Management Interview

A management interview was conducted by telephone conference call on June 3, 1974, with the following persons.

#### Jersey Central Power and Light

Mr. J. T. Carroll, Station Superintendent  
Mr. K. O. Fickerssen, Jr., Technical Supervisor  
Mr. E. J. Gowney, Technical Engineer

The following summarizes the item discussed.

#### A. Containment Integrated Leak Rate Test

The licensee stated that he has revised the CILRT procedure in areas of concern documented in report 50-219/74-09. Areas reviewed by inspector were:

1. Reassignment of Instrument Volume Fractions. (Details, Paragraph 3.a);
2. Calculated Containment Free Volume. (Details, Paragraph 3.b);
3. Temperature Stabilization Criteria. (Details, Paragraph 3.c);
4. 10 CFR 50, Appendix J requirements. (Details, Paragraph 3.d);
5. Containment Test Format. (Details, Paragraph 3.e).

B. In Core Flux Monitor Housing (AO 50-219/74-34)

The licensee stated that a program to determine the cause and institute repair of the In-Core flux monitor housing leakage has not been developed.

The licensee stated that, prior to resumption of power operation, report will be submitted to the Directorate of Licensing. (Details, Paragraph 4)

C. Control Rod Drive Stub Tube Examination

The inspector stated that he had witnessed removal of the CRD mechanism, core position 18-47, and of the thermal sleeve, in preparation for ultrasonic testing of the weld joint. (Details, Paragraph 5)

## DETAILS

### 1. Persons Contacted

Mr. R. M. Bright, Associate Engineer  
Mr. J. T. Carroll, Station Superintendent  
Mr. R. Dube, Quality Assurance Supervisor  
Mr. K. O. Frickeissen, Jr., Technical Supervisor  
Mr. E. J. Gowney, Technical Engineer  
Mr. D. L. Reeves, Jr., Chief Engineer  
Mr. E. Rosenfeld, Associate Engineer  
Mr. D. A. Ross, Manager, Nuclear Generating Station  
Mr. J. L. Sullivan, Jr., Operations Engineer

### 2. General

The inspector was notified by the licensee of the pending Containment Integrated Leak Rate Test (CILRT), and proceeded to the site on May 30, 1974. The licensee reported the Abnormal Occurrence 50-219/74-34 and postponed indefinitely the CILRT.

### 3. Containment System Leak Rate Test

The licensee notified the inspector that the local leak rates have not been completed. The inspector reviewed the revision of CILRT procedure and closed out the following areas of concern:

#### a. Reassignment of Instrument Volume Fractions

The licensee's Containment Integrated Leak Rate Test presently attributes equal volume fractions to the temperature and dewpoint test instruments. Upon loss of an instrument the reassigned values of volume fraction will be equally shared by the remaining operable instruments. The licensee has also established a criterion on the minimum number of operable instruments. This concern, documented in Report 50-219/74-09, Details, Paragraph 2.b, is resolved.

#### b. Calculated Containment Free Volume

The licensee presented to the inspector the calculations of containment free air volume and torus water level as a function of air volume. This concern, documented in Report 50-219/74-09, Details, Paragraph 2.c, is resolved.

c. Temperature Stabilization Criteria

The licensee has established, in the CILRT procedure, a stabilization criteria of 1°F/hr rate of change of average temperature over a two hour period. This area of concern, documented in Report 50-219/74-09, Details, Paragraph 2.d, remains unresolved pending resolution of the 0.1°F/hr criterion referred to RO:HQ.

d. Appendix J of 10 CFR 50, Additional Requirements

The licensee has revised the CILRT procedure to reflect the requirements of 10 CFR 50, Appendix J in the area of testing. This concern, documented in Report 50-219/74-09, Details, Paragraph 2.g, is resolved.

e. Integrated Primary Containment Leak Rate Test - Format

The licensee has revised the CILRT to include the following.

1. Signature and date block entry requirements for the prerequisites and performance of the test.
2. System diagram reference numbers have been added to the procedure.
3. Elimination of non-definitive statements and referenced operational procedures where necessary.

This concern, documented in Report 50-219/74-09, Details, Paragraph 2.h, is resolved.

4. In Core Flux Monitor Housing Abnormality

The licensee Abnormal Occurrence report 50-219/74-34 dated May 30, 1974, identifies coolant leakage in the area of the In Core Flux monitor reactor vessel housing at core coordinate 28-05. Leakage was found during scheduled reactor vessel hydrostatic test inspection of pressure boundaries following a routine refueling maintenance activity.

The licensee has contacted the nuclear steam supplier and reactor vessel manufacturer. Onsite discussions to determine the course of action were in progress during the inspection.

The licensee stated that the evaluation of the problem is not complete at present time, and that RO:HQ and RO:I will be informed of the progress of discussions.

The licensee also stated that, prior to returning to power operations, he would submit a report for review and approval by the Directorate of Licensing.

5. Control Rod Drive - Position 18-47

The licensee, upon identification of the In-Core Flux Tube leakage, initiated reinspection of all bottom head reactor penetrations. During this inspection, discoloration was noted on the Control Rod Drive Mechanism housing at core position 18-47.

The inspector witnessed portions of the CRD mechanism removal from the reactor vessel, and thermal sleeve removal. The licensee was preparing to perform an ultrasonic test inspection of the weld area between the stub tube and CRD sleeve.

The licensee reported to the inspector by telephone that no abnormalities were determined. This item will be subsequently reviewed.