PUBLIC SERVICE COMPANY OF COLORADO

FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 132

January, 1985

IE24

1/1

This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of January, 1985.

1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The reactor was depressurized for the month, and decay heat removal was being performed by "1B" Helium Circulator and the Loop I Steam Generator. Helium Circulator "1A" was removed and shipped to San Diego to repair an interspace bearing water leak. The circulator has not been returned due to problems noted during reassembly. The problems are being thoroughly analyzed and repaired. The arrival date for the circulator has not been determined.

The Loop II cold reheat Marmon flanges have all been replaced with spoolpieces. The remaining work on the Loop II reheat flanges is to seal weld the upper Marmon flanges. This will start after the flange plugs are machined.

The circulating water tower has been isolated and drained to prevent icing, due to the low heat load during the present plant shutdown conditions. The Steam Jet Air Ejector (SJAE) has been isolated, and has been checked for proper jet size and water leaks.

A concentrated effort was made to analyze System 21, Helium Circulator Bearing Water Pump problems. Pump, P-2101, was repaired and carefully analyzed to try to determine the causes of our recent bearing water pump problems. Initial indications were that the base was not level and that piping induced stresses on the pump during operation. The pump was returned to service on January 31, 1985. Vibration was approximately one mil, and the pump had a small leak. The pump and motor were shutdown, decoupled, the leak was fixed, and the pump was returned to service. The results of this rebuild will be evaluated after the pump has operated for some time. When the pump was returned to service, Helium Circulator "1B" tripped due to air accumulation in the pump and lines. The circulator was restarted with no complications. Later, another restart of P-2101 caused another helium circulator trip.

The refueling floor has been cleared of all items not required for the control rod drive rework program. The temporary ten ton crane has been installed. The refueling floor and hot service facility are being modified with special equipment items necessary to implement the control rod drive refurbishment program. This program is scheduled to last approximately two months.

The Essential Bus 480V Breakers were all checked for their annual inspection. Setpoint problems were noted on the first breaker tested, but no other breakers have shown a setpoint drift. This activity is essentially complete.

"1B" Instrument Air Compressor five year preventive maintenance inspection and overhaul has been completed and the compressor has been tested, passed, and returned to service. Operational problems developed and the compressor had to be shutdown for valve replacment. The compressor is now out of service.

A major leak occured on "A" Condensate Pump. That leak has been repaired. However, when trying to establish pump clearance, several condensate valves where found to be leaking, and are now being repaired.

Primary coolant circulation was terminated on January 21, 1985, to repair the leaking condensate header valves. All but one valve was returned to service on January 26, 1985. Helium Circulator "1B" was restarted for primary coolant circulation. No secondary coolant was established. This arrangement is maintaining primary coolant temperatures between 170 and 200 degrees Fahrenheit. Secondary coolant will be established after the remaining valve is completed, and before the control rods are removed from the core, if necessary.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None

4.0 MONTHLY OPERATING DATA REPORT

Attached

OPERATING DATA REPORT

DOCKET NO. 50-267
DATE February 5, 1985
COMPLETED BY Frank Novachek
TELEPHONE (303) 785-2224

OPERATING STATUS

NOTES

1. Unit Name: Fort St. Vrain
2. Reporting Period: 850101 through 850131
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   None
9. Power Level To Which Restricted, If Any (Net MWe): 280
10. Reasons for Restrictions, If Any: Per commitment to the NRC, long term operation above 85% power is pending completion of B-O startup testing.

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	744	49,009
12. Number of Hours Reactor Was Critical	0.0	0.0	27,151.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	0.0	18,468.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	9,861,714.4
17. Gross Electrical Energy Generated (MWH)	0	0	3,248,594
18. Net Electrical Energy Generated (MWH)	-2,017	-2,017	2,925,941
19. Unit Service Factor	0.0	0.0	37.7
20. Unit Availability Factor	0.0	0.0	37.7
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	18.1
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	18.1
23. Unit Forced Outage Rate	100.0	100.0	47.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 850201 through 850331. 1,416 hours, Control Rod Drive Refurbishment.
25. If Shut Down at End of Report Period, Estimated Date of Startup: 850401
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain #1

Date February 5, 1985

Completed By Frank Novachek

Telephone (303) 785-2224

Month January, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	0.0
16	0.0		

*Generator on line but no net generation.

50-267

UNIT NAME Fort St. Vrain #1

DATE February 5, 1985

COMPLETED BY Frank Novachek

TELEPHONE (303) 785-2224

REPORT MONTH January, 1985

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
84-006	850101	F	744.0	A	3	50-267/84-008	AA	JC	Control Rod Drive Investigation

REFUELING INFORMATION

1. Name of Facility	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	4th Refueling: February 1, 1986
3. Scheduled date for restart following refueling.	May 1, 1986
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	No
If answer is yes, what, in general, will these be?	----------
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	No
If no such review has taken place, when is it scheduled?	1985
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	----------
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	----------
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	a) 1482 HTGR fuel elements b) 143 spent fuel elements

REFUELING INFORMATION (CONTINUED)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

* The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accomodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.

Public Service Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

February 14, 1985
Fort St. Vrain
Unit #1
P-85052

Office of Inspection and Enforcement
ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket No. 50-267

REFERENCE: Facility Operating License No. DPR-34

SUBJECT: Monthly Operations Report-January, 1985

Dear Sir:

Enclosed please find our Monthly Operations Report for the month of January, 1985.

Sincerely,

J. W. Gahm
Manager, Nuclear Production

Enclosure

cc: Mr. John T. Collins

JWG/djm

IE24
1/1
ORIG. TO REGION IV