NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Susquehanna Steam Electric Station - Unit 2	0 5 0 0 0 3 8 8	1 OF 0 2

TITLE (4)

ESF Actuation - RWCU Isolation Valves Closed.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	1 4	8 5	8 5	0 0 4	0 0	0 2	1 2	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
L.A. Kuczynski - Nuclear Plant Specialist, Level III	AREA CODE 7 1 7 5 4 2 - 3 7 5 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	K M	F S	B 0 7 4	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

The Reactor Water Cleanup System (RWCU) had been experiencing isolations due to non-regenerative heat exhanger high outlet temperature. On January 14, 1985, in an effort to prevent another RWCU isolation, Operations personnel throttled opened the RWCU filter demineralizer bypass valve. This resulted in closure of the RWCU inboard and outboard isolation valves on a high system flow signal. These valves are part of the Primary Containment Isolation System, which is an Engineered Safety Feature. The RWCU isolations had been occurring as part of a sequence cf events triggered by the trip of the Reactor Building Chilled Water (RBCW) system. The RBCW trips were traced to a loose connection in an RBCW flow switch. The connection was tightened. There have been no recurrences. No further action is required.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Susquehanna Steam Electric Station Unit 2	0 5 0 0 0 3 8 8	8 5	— 0 0 4	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On January 14, 1985, in response to a Reactor Building Closed Cooling Water (RBCCW) isolation to the Reactor Water Cleanup (RWCU) non-regenerative heat exchangers (NRHX), Operations personnel (licensed, utility) attempted to avert an imminent RWCU isolation by throttling open the RWCU filter/demineralizer bypass valve. The bypass valve was opened somewhat too quickly, and caused a high system flow signal which resulted in the closure of the RWCU inlet inboard and outboard isolation valves. These valves are part of the Primary Containment Isolation System, which is an Engineered Safety Feature. The RWCU system was quickly restored to service.

Beginning on January 4, 1985, problems were experienced with the Reactor Building Chilled Water (RBCW) system. An RBCW low flow switch would send a false low flow signal to the Reactor Building Closed Cooling Water (RBCCW) system. This would cause RBCCW to shed its normal loads (which includes the RWCU NRHX) to pick up the RBCW load of the drywell coolers. This led to several RWCU isolations on RWCU NRHX high outlet temperature. This is a non-safety related system isolation, designed to protect the resin in the RWCU filter/demineralizers. Investigation by Instrumentation and Controls personnel found a loose connection in the terminal box for the RBCW low flow switch. The connection was tightened and the system was monitored for eight days with no recurrences. No further action is required.

PP&L Pennsylvania Power & Light Company
Two North Ninth Street • Allentown, PA 18101 • 215/770-5151

February 12, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 85-004-00
ER 100450 FILE 841-23
PLAS-039

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 85-004-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the Unit experienced an unanticipated actuation of an Engineered Safety Feature when the Reactor Water Cleanup System inlet inboard and outboard isolation valves closed on a high flow signal.

H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

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