

To:

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From:

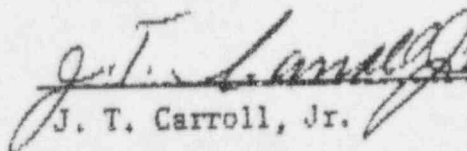
Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-219
Forked River, New Jersey 08731

Subject:

Abnormal Occurrence Report No. 50-219/74/ 13

The following is a preliminary report being submitted
in compliance with the Technical Specifications
paragraph 6.5.2.

Preliminary Approval:


J. T. Carroll, Jr. Date 2/19/74

cc: Mr. A. Giambusso

B/628

cleanup isolation signal should have been present at this time due to a high pressure condition on pressure switch ⁸¹⁰⁴. This switch, set to actuate at 140 psig, is located upstream of the pressure relief valve. Ordinarily isolation valves V-16-1 and V-16-14 would have gone closed under these circumstances. However, since the breaker to V-16-1 was open, closure was impossible. Valve V-16-14 failed to close automatically and an unsuccessful attempt was made to close it by placing the valve position selector switch to the close position.

Approximately five minutes after returning the selector switch for V-16-14 to the open (automatic closure) position, the valve closed. The breaker for V-16-1 was reclosed and valve closure was achieved when the selector switch was placed in the close position.

APPARENT CAUSE
OF OCCURRENCE:

<input type="checkbox"/>	Design	<input type="checkbox"/>	Procedure
<input type="checkbox"/>	Manufacture	<input type="checkbox"/>	Unusual Service Condition
<input type="checkbox"/>	Installation/ Construction	<input type="checkbox"/>	Inc. Environmental Component Failure
<input type="checkbox"/>	Operator	<input type="checkbox"/>	Other (Specify)

The cause of Valve V-16-1 failing to isolate is attributable to the inadvertent opening of its associated breaker. The cause for V-16-14 failing to isolate is presently under investigation.

Initial Written
Report Date: 2/19/74

Time of
Occurrence: 1120

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/13

IDENTIFICATION
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 3.5.A.1, when the cleanup system AC isolation valve V-16-1 became inoperable with the reactor critical and the coolant temperature above 212°F. In addition, violation of the Technical Specification Table 3.1.1-F.2, when the cleanup system DC isolation valve V-16-14 failed to close when called upon to do so.

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraphs 1.15B & D.

CONDITIONS PRIOR
TO OCCURRENCE:

<u>X</u>	Steady State Power	_____	Routine Shutdown
_____	Hot Standby	_____	Operation
_____	Cold Shutdown	_____	Load Changes During
_____	Refueling Shutdown	_____	Routine Power Operation
_____	Routine Startup	_____	Other (Specify)
_____	Operation	_____	

Power: Reactor, 1895 MWt
Elec., 671 MWe
Flow: Recirc., 57.6×10^6 lb/hr
Feed., 7.08×10^6 lb/hr
Stack Gas: 29,329 μ Ci/sec

DESCRIPTION



On Monday, February 18, 1974, at 1120, the breaker for the

cleanup system automatic isolation valve V-16-1 was accidentally tripped which consequently caused the cleanup recycle pump NDO2A to trip. As pressure increased in the system piping, due in part to pressure reducing valve ND-11 failing to control pressure, the pressure downstream of ND-11 increased above the cleanup to torus relief valve setpoint of 150 psig. An automatic

ANALYSIS OF
OCCURRENCE:

Since both cleanup system isolation valves were inoperable or malfunctioned during this event had a loss of coolant accident occurred generating a Lo-Lo reactor water level condition, the cleanup system would not have isolated. This is only significant if there is also a leak in the cleanup system loop in which case a non-isolable leak of reactor coolant into the secondary containment would have been created. It should be noted that after reclosing the breaker for V-16-1 isolation would have occurred.

CORRECTIVE
ACTION:

The breaker for Valve V-16-1 was reenergized, thereby reestablishing its isolating capabilities. No corrective action was taken for V-16-14 as it appeared to be functioning normally.

FAILURE DATA:

Not applicable.

Prepared by:

Arthur H. Fone

Date:

2/19/74