


To: James P. O'Reilly  
Directorate of Regulatory Operations  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

From: Jersey Central Power & Light Company  
Oyster Creek Nuclear Generating Station Docket #50-219  
Forked River, New Jersey 08731

Subject: Abnormal Occurrence Report No. 50-219/74/ 5

The following is a preliminary report being submitted  
in compliance with the Technical Specifications  
paragraph 6.6.2.

Preliminary Approval:

  
\_\_\_\_\_  
J. T. Carroll, Jr.                      Date                      1/17/74

cc: Mr. A. Giambusso

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OYSTER CREEK NUCLEAR GENERATING STATION  
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence  
Report No. 50-219/74/ 5

IDENTIFICATION  
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 4.5.F.1.d, failure of Main Steam Isolation Valve NS04A to meet the allowable leakage requirements.

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15E.

CONDITIONS PRIOR  
TO OCCURRENCE:

- |   |  |
|---|--|
| <input type="checkbox"/> Steady State Power       | <input type="checkbox"/> Routine Shutdown        |
| <input type="checkbox"/> Hot Standby              | <input type="checkbox"/> Operation               |
| <input checked="" type="checkbox"/> Cold Shutdown | <input type="checkbox"/> Load Changes During     |
| <input type="checkbox"/> Refueling Shutdown       | <input type="checkbox"/> Routine Power Operation |
| <input type="checkbox"/> Routine Startup          | <input type="checkbox"/> Other (Specify)         |
| <input type="checkbox"/> Operation                |  |

The plant was shutdown with the reactor coolant at  $<212^{\circ}\text{F}$ .

DESCRIPTION  
OF OCCURRENCE:

1447 - Leak rate test on Main Steam Isolation Valve NS04A began.  
1517 - Leak rate test on Main Steam Isolation Valve NS04A ended.

Leakage rate was equivalent to 31.7 SCFH. The maximum allowable leakage rate is 9.945 SCFH, as required by Technical Specification, paragraph 4.5.F.1.d.

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APPARENT CAUSE  
OF OCCURRENCE:

Design  
 Manufacture  
 Installation/  
 Construction  
 Operator

Procedure  
 Unusual Service Condition  
 Inc. Environmental  
 Component Failure  
 Other (Specify)

After checking the test assembly and the components of the MSIV, it was determined that the packing around the valve shaft was the cause of the excessive leak rate.

ANALYSIS OF  
OCCURRENCE:

The safety significance of the failure of NS04A is dependent on the condition of the inside valve NS03A. NS03A was considered to have an acceptable leakage rate (2.0 SCFH) when tested on January 13, 1974. However, the validity of the test is questionable since the testing procedure assumes minimal leakage through NS04A. The safety significance will be evaluated following a retest of NS03A.

CORRECTIVE  
ACTION:

The valve shaft was repacked and the valve was retested successfully. NS03A will be retested to insure that it has an acceptable leak rate.

FAILURE DATA:

The valve stem packing on NS04A failed on September 27, 1973. This valve was repacked and passed its leak rate test.

Prepared by:

Charles S. Crozney

Date:

1/17/74