

UNITED STATES
ATOMIC ENERGY COMMISSION
DIVISION OF COMPLIANCE
REGION I
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May 12, 1972

R. T. Carlson, Sr. Reactor Inspector
Regulatory Operations, HQ

A handwritten signature, likely of R. T. Carlson, Sr., is written in blue ink to the right of the recipient's name.

RO INSPECTION REPORT NO. 50-219/72-02
JERSEY CENTRAL POWER & LIGHT COMPANY
OYSTER CREEK - BWR

If there is one thing I have learned from this inspection, it is that I am not going to permit seven months to pass again without a routine inspection. There are too many items that can't get proper attention. For example, my inspection list included 35 items before I entered the plant and started reading the log books. Something is lost in our inspection program when this amount of time elapses between routine inspections or this number of items is allowed to develop. Of course, there were four special inspections, but these were problem oriented inspections of limited scope.

The requested action or information was obtained for the following HQ memos:

1. Torus to Drywell Vacuum Breaker Check Valves for BWRs, 1/6/72 - Information forwarded by separate memo.
2. Millstone Point Company Report to DRL Describing Failure Resulting in Loss of Automatic Blowdown Function for a Relief Valve, 1/12/72 - Licensee was informed of the problem but does not have Target Rock relief valves.
3. Monticello Unit 1 - CO Inspection Report No. 263/71-13, 1/17/72 - A circuit review by the maintenance supervisor showed that the off gas isolation system will not reset automatically.
4. Monticello - Differential Pressure Switch Failure, 1/12/72 - Informed the maintenance supervisor of the contamination problem on the jewel bearing in the DP switch. No reported failures at Oyster Creek.


5. Niagara Mohawk Power Corporation - Report to DRL, December 13, 1971, 1/11/72 - Licensee plans to inspect the main steam flow venturi pressure taps during the April 1972 outage.
6. Monticello - Licensee Report to DRL Describing Removal of Baffles from Torus and Modification to Relief Valve Discharge Piping, 1/12/72

The licensee was informed of the problem and plans to inspect the torus at least above the water line during the April 1972 refueling outage. Due to the chromate in the torus water, they do not plan to drain the torus unless the above water inspection indicates problems.
7. Reactor Pressure Vessel Thermal Cycles, 1/11/72 - All pressure and temperature data has been accumulated in one log. Usage data for some of the more important parameters has been calculated. OC plans to review and update this data at approximately six month intervals.
8. Oyster Creek - Isolation Relay Failure, 9/27/71 - The inspector was informed that the specific relay in question was a basic GECR120 relay that was modified to make it a CR122A-09041AA time delay relay. These relays have been replaced.
9. Emergency Service Water Pumps, 10/27/71 - Pumps are tested monthly, but do not have any performance specifications. Will establish standard test pressure but can not measure flow.
10. Reporting Abnormal Occurrences, 8/17/71 - The subject has been discussed several times, and the licensee is being cited for failure to promptly report an abnormal occurrence.

I was disappointed in the report of the Loss of 4160 V Emergency Bus 1C (Letter to DRL, March 10, 1972); however, in my telephone discussion with Ivan Finrock, I feel that I got a commitment to expand the use of procedures in performing safety related maintenance work. (Inquiry Report 219/72-06, dated 4/12/72 gives another example of the problems that occur due to the failure to use written procedures.) I expect a dramatic change in JCP&L's attitude toward written procedures.

As a result of the review of the loss of the 1C emergency bus, it appears the TS permit both a startup transformer (supplying off site power) and an emergency diesel generator to be out of service at the same time without any cross restriction on the availability of the other. I question whether this is the intent of the TS.

I am planning to go back to Oyster Creek April 21, 1972 to witness their full load turbine trip test and observe reactor startup as part of the operator licensing examination. This inspection will be limited to these two items.



F. S. Cantrell
Reactor Inspector