

April 15, 1996

Mr. Donald Schnell
Senior Vice President - Nuclear
Union Electric Company
Post Office Box 149
St. Louis, Missouri 63166

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING UNION ELECTRIC COMPANY'S MARCH 12, 1996, REQUEST FOR RELIEF TO THE CALLAWAY PLANT INSERVICE INSPECTION PROGRAM PLAN (TAC NO. M94961)

Dear Mr. Schnell:

The NRC staff has reviewed Union Electric Company's March 12, 1996, request for relief to the Callaway Plant Inservice Inspection Program Plan. As a result of the review, the staff has determined that additional information is needed to complete the review. The information needed is detailed in the enclosure.

To assist the NRC staff in meeting its review schedule, we request that you respond to the RAI in writing within 30 days of receipt of this letter.

If you have any questions, please contact me at (301) 415-1362.

Sincerely,

Original Signed By

Kristine M. Thomas, Project Manager
Project Directorate IV-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-483

Enclosure: Request for Additional Information

cc w/encl: See next page

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April 15, 1996

cc w/encl:

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REQUEST FOR ADDITIONAL INFORMATION

CALLAWAY PLANT, UNIT 1

REQUEST FOR RELIEF E-6 TO THE INSERVICE INSPECTION PROGRAM PLAN

Effective September 8, 1992, regulations were issued regarding the augmented examination of reactor vessels. As a result of these regulations, all licensees must augment their reactor vessel examinations for the inservice inspection interval in effect on September 8, 1992, by implementing, once, the examination requirements for reactor vessel shell welds specified in Item B1.10 of Examination Category B-A. In addition, all previously granted reliefs for Item B1.10, Examination Category B-A, for the interval in effect on September 8, 1992, were revoked by this regulation. For licensees with fewer than 40 months remaining in the interval on the effective date, deferral of the augmented examination is permissible with the conditions stated in the regulations.

Please provide the staff with a technical discussion describing how the regulation has been or will be implemented for the reactor pressure vessel (RPV) shell welds at the Callaway Plant, Unit 1. Include a description of the approach and any specialized techniques or equipment that have been or will be used to complete the required augmented examination. If this augmented examination has already been performed, verify that at least 90 percent of each RPV shell weld (Items B1.11 and B1.12) has been examined or propose an alternative to the examination requirements as specified by 10 CFR 50.55a(g)(6)(ii)(A)(5). For relief request E-6, quantitative information on the expected coverage increase from supplemental hand scans, outside-diameter examinations, and the design and use of specialized tooling for the three Item B1.12 RPV upper-shell longitudinal welds is needed.