

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293  
UNIT Pilgrim 1  
DATE February 13, 1985  
COMPLETED BY P. Hamilton  
TELEPHONE (617)746-7900

MONTH January 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>97.</u>	17	<u>314.</u>
2	<u>0.</u>	18	<u>428.</u>
3	<u>0.</u>	19	<u>600.</u>
4	<u>0.</u>	20	<u>652.</u>
5	<u>0.</u>	21	<u>644.</u>
6	<u>0.</u>	22	<u>659.</u>
7	<u>0.</u>	23	<u>657.</u>
8	<u>0.</u>	24	<u>537.</u>
9	<u>23.</u>	25	<u>512.</u>
10	<u>164.</u>	26	<u>666.</u>
11	<u>182.</u>	27	<u>667.</u>
12	<u>285.</u>	28	<u>640.</u>
13	<u>293.</u>	29	<u>494.</u>
14	<u>316.</u>	30	<u>668.</u>
15	<u>461.</u>	31	<u>670.</u>
16	<u>474.</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

8502210296 850131  
PDR ADOCK 05000293  
R PDR

IE24  
1/1

OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE February 13, 1985  
 COMPLETED BY P. Hamilton  
 TELEPHONE (617)746-7900

OPERATING STATUS

1. Unit Name Pilgrim 1 Notes  
 2. Reporting Period January 1985  
 3. Licensed Thermal Power (Mwt) 1998  
 4. Nameplate Rating (Gross MWe) 678  
 5. Design Electrical Rating (Net MWe) 655  
 6. Maximum Dependable Capacity (Gross MWe) 683  
 7. Maximum Dependable Capacity (Net MWe) 663  
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
None

9. Power Level To Which Restricted, If Any (Net MWe) None  
 10. Reasons For Restrictions, If Any N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>106488.0</u>
12. Number Of Hours Reactor Was Critical	<u>600.0</u>	<u>600.0</u>	<u>70516.6</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>557.0</u>	<u>557.0</u>	<u>68125.5</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
* 16. Gross Thermal Energy Generated (MWH)	<u>822504.0</u>	<u>822504.0</u>	<u>117774312.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>278060.0</u>	<u>278060.0</u>	<u>39510274.0</u>
18. Net Electrical Energy Generated (MWH)	<u>266497.0</u>	<u>266497.0</u>	<u>37963424.0</u>
19. Unit Service Factor	<u>74.9</u>	<u>74.9</u>	<u>64.0</u>
20. Unit Availability Factor	<u>74.9</u>	<u>74.9</u>	<u>64.0</u>
21. Unit Capacity Factor (Using MDC Net)	<u>54.0</u>	<u>54.0</u>	<u>53.8</u>
22. Unit Capacity Factor (Using DER Net)	<u>54.7</u>	<u>54.7</u>	<u>54.4</u>
23. Unit Forced Outage Rate	<u>25.1</u>	<u>25.1</u>	<u>9.4</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup -  
 26. Units In Test Status (Prior to Commercial Operation):

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	<u>_____</u>	<u>_____</u>
INITIAL ELECTRICITY	<u>_____</u>	<u>_____</u>
COMMERCIAL OPERATION	<u>_____</u>	<u>_____</u>

(9/77)

\* - This item was misstated in the December 1984 Operating Data Report. The correct figures for December are 19176.0 Mwt for the month, 19176.0 Mwt for the year-to-date, and 116951808.0 Mwt cumulative.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293  
 UNIT NAME Pilgrim 1  
 DATE February 13, 1985  
 COMPLETED BY P. Hamilton  
 TELEPHONE (617) 746-7900

REPORT MONTH January 1985

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	1/1/85	F	187.0	H	1	85-001-00	BR	N/A	Cause of the SLCS Inop due to debris in system. Corrective action was to drain, clean, and/or flush the system.
2	1/17/85	S	0.0	B	N/A	N/A	N/A	N/A	N/A - Power reduced for testing and maintenance.
3	1/29/85	F	0.0	B	N/A	N/A	N/A	N/A	N/A - Power reduced for maintenance.

1	2	2	3	4 & 5
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)

## REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next Refueling Shutdown: August 1986
3. Scheduled date for restart following refueling: November 1986
- 4.
5. Due to their similarity, requests 4, 5, & 6 are responded to collectively:
6. The new fuel, which is being loaded this refueling outage, is of the same P8x8R design, as loaded the previous outage and consists of 160 P8DRB282 assemblies and 32 GE6B-P8DRB282 assemblies.
7. (a) There are 580 fuel assemblies in the core.  
(b) There are 1,128 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual spent fuel storage capacity is 1770 fuel assemblies at present.  
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 642 fuel assemblies.

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET NO. 50-293

Operational Summary for January 1985

The unit was removed from service on the first of the month as a result of problems with the standby liquid control system. During the shutdown, work was also performed on the main condenser and the drywell to torus vacuum breakers.

The reactor was made critical on the seventh, at which time the head spray check valve was found to be leaking. The valve was repaired and, on the ninth, the generator was synchronized to the grid.

Between the twelfth and the eighteenth, power was maintained at an average daily level between 40% and 70% to facilitate operational and reactor engineering testing. Subsequent to the tests, power was reduced to approximately 38% to verify jet pump flow data and repaired the "A" feedwater regulating valve.

Between the nineteenth and the twenty-eighth, the average daily power level ranged between 76% and 98%. During this time frame, both recirculation pumps were tripped for testing.

On the twenty-ninth, power was reduced to approximately 74% due to a high differential temperature between cooling water intake and discharge.

On the thirtieth, after resolution of the high differential temperature, power was increased to 100% and maintained at that level through the end of the month.

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Safety Relief Valve Challenges

Month of January 1985

Requirement: NUREG-0737

T.A.P.

II.K.3.3

Date: January 8, 1985 (Manually Opened Twice)

Valve #: RV-203-3A

Reason: Check High Tail Pipe Temperature  
Test New Solenoid Valve

Month January 1985

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
SLCS	Relief Valve	System declared inoperable.	Debris in System	Drained, cleaned, and/or flushed, the system.	Locked/Bolted SLCS Tank Covers	85-001-00
HPCI	2301-3 Steam Admission Valve	Would not close during surveillance.	Limit switch out of adjustment.	Adjust limit switch.	N/R	N/R
HPCI	Y Strainer 8048	Leaking	Bad Gasket	Replaced gasket.	N/R	N/R
RPS	PS-1001-23A High Drywell Pressure Switch	Actuating lever misaligned.	Faulty	Repaired	N/R	N/R
RPS	LIS-263-73B Reactor Water Level Indication Switch	Out of calibration.	Probable setpoint drift.	Recalibrated	N/R	N/R
RPS	Main Steam Line Low Pressure Switches PS-261-30A&D	Out of calibration.	Setpoint Drift	Recalibrated	Increased calibration surveillance frequency.	N/R
Main Steam	Safety Relief Valve	High Tail Pipe Temp.	Faulty Solenoid	Replaced solenoid valve.	Replaced solenoid valve.	N/R
RHR	Head Spray Check Valve	Leaking slightly.	Body To Bonnet Flange Leak	Furmanited	N/R	N/R

BOSTON EDISON COMPANY  
800 BOYLSTON STREET  
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON  
SENIOR VICE PRESIDENT  
NUCLEAR

February 13, 1985  
BECo Ltr. #85-032

Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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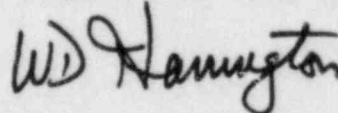
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Docket No. 50-293

Subject: January 1985 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning.

Respectfully submitted,



W. D. Harrington

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:caw

Attachment

cc: Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

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