

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE February 6, 1985
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: January, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>99,530.0</u>
12. Number Of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>76,024.2</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>75,411.4</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,108,294.9</u>	<u>1,108,294.9</u>	<u>95,295,061.9</u>
17. Gross Electrical Energy Generated (MWH)	<u>377,374.0</u>	<u>377,374.0</u>	<u>31,146,999.0</u>
18. Net Electrical Energy Generated (MWH)	<u>360,547.8</u>	<u>360,547.8</u>	<u>29,772,185.1</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>75.8</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>75.8</u>
21. Unit Capacity Factor (Using MDC Net)	<u>101.4</u>	<u>101.4</u>	<u>65.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>101.4</u>	<u>101.4</u>	<u>62.9</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>3.8</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1985

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE February 6, 1985
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No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or power reductions during the month of January, 1985.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending January, 1985 .

- 1. Scheduled date for next refueling shutdown. October 1985
- 2. Scheduled date for restart following refueling. December 1985
- 3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

a. If answer is yes, what, in general, will these be?

Technical Specification change to accommodate increased radial peaks due to further reduction in radial leakage.

b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

c. If no such review has taken place, when is it scheduled?

- 4. Scheduled date(s) for submitting proposed licensing action and support information. September 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Methodology Changes June 1985

- 6. The number of fuel assemblies:

a) in the core	133	assemblies
b) in the spent fuel pool	305	"
c) spent fuel pool storage capacity	729	"
d) planned spent fuel pool storage capacity	May be increased via fuel pin consolidation	"

- 7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by JR Laper Date February 4, 1985

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Station
 DATE February 6, 1985
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

MONTH January, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	482.0	17	485.6
2	482.6	18	485.2
3	482.9	19	484.6
4	483.7	20	484.8
5	486.2	21	484.2
6	486.2	22	483.3
7	484.7	23	484.6
8	483.7	24	484.4
9	483.7	25	483.7
10	484.0	26	484.3
11	483.2	27	484.7
12	484.4	28	484.1
13	486.9	29	484.7
14	486.1	30	485.4
15	486.1	31	486.4
16	486.6		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

January, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power throughout January, 1985. A Reactor Operator was promoted to Shift Supervisor effective January 16, 1985. A Shift Supervisor transferred to the Training Department.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
84-008	Steam Generator Tube Rupture.
84-014	VIAS Actuation (RM-060).
84-015	Load Over the RCS.
84-024	VIAS Actuation (RM-061).
84-023	VIAS Actuators (RM-060, RM-050)
84-025	RM-060 VIAS Actuation.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-BURNUP-1	Burnup Determination for Storage of Spent Fuel. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it verifies that burnup criteria are met for spent fuel storage as required in the NRC approved spent fuel rack installation.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.
SP-EEQ-1	LOCA Qualified Electrical Equipment Identification Check. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure is simply an inventory list of qualified equipment and indications located on various panels and control boards in the plant. No plant operations or evolutions are involved.
SP-SI-4	Loop Valve Setting Procedure. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it was performed in accordance with an approved procedure and accomplished the desired results.
SP-SOV-1	Periodic Cycling of Solenoid Valves Preventive Maintenance to Maintain 79-01B Qualification. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure aids in assuring all valves needed to attain hot shutdown following an accident are operable. This is accomplished through periodic cycling via the qualified life program.

System Acceptance Committee Packages for January, 1985:

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-83-33	TSC Room Modification. This modification provided for the installation of a sliding glass door between Room 107 and Room 115 in the TSC and has no adverse effect on the safety analysis.

D. CHANGES, TEST AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

System Acceptance Committee Packages for January, 1985: (continued)

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-83-68	Installation of LCV-1196 and 1197 Valves. This modification did not effect safety related equipment, therefore, has no adverse effect on the safety analysis.
EEAR FC-83-82	Auxiliary Building Communication. This modification did not effect safety related equipment, therefore, has no adverse effect on the safety analysis.
EEAR FC-84-53	Automatic Rod Withdrawal Alarm. This modification provided for the deletion of this alarm. This alarm is considered a nuisance alarm since the rods are never controlled in the automatic mode and are never planned to be. Also, the three conditions that give this alarm are individually annunciated on three separate windows. This modification has no adverse effect on the safety analysis.
EEAR FC-84-151	Diesel No. 1 Solenoid Valve Replacement. This modification provided for a one-to-one replacement of solenoid valves and rotated control levers on associated air motors. This will provide for correct operation of the intake dampers. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

Procedure ST-CONT-2, Section F.2, was completed on January 23, 1985. This is the six month Personnel Air Lock Leak Rate Test. The new as found leak rate increased the total Type B and C Local Leak Rate from 10,139.73 sccm to 11,749.73 sccm. This is well within the .6 La specified in Appendix J. 10CFR50.

F. CHANGES IN PLANT OPERATING STAFF

Mr. James B. Michael was promoted to Shift Supervisor effective January 16, 1985. Mr. George J. Pelnar transferred from the Operations Department as Shift Supervisor to the Training Department.

G. TRAINING

Operators received extensive fire brigade leader training in addition to preplanned lecture series. A Self Evaluation Report (SER) was submitted to the Institute of Nuclear Power Operation (INPO) for the operator non-licensed, initial licensed and licensed requalification programs. This is three of ten programs that will be certified and accredited by INPO.

Training was conducted for Quality Control personnel on non-destructive testing. Training was also conducted in the areas of General Employee Training (Initial and Requalification), systems and emergency plan for plant personnel.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates
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Manager
Fort Calhoun Station