



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 12, 1996

Mr. J. T. Beckham, Jr.
 Vice President - Plant Hatch
 Georgia Power Company
 P.O. Box 1295
 Birmingham, Alabama 35201

SUBJECT: SAFETY EVALUATION OF THIRD TEN-YEAR INTERVAL FOR THE PUMP AND VALVE INSERVICE TESTING PROGRAM - EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 - (TAC NOS. M93072 AND M93073)

Dear Mr. Beckham:

By letter dated September 15, 1995, you submitted the Third Ten-Year Interval Inservice Testing (IST) Program for Pumps and Valves for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The NRC staff has reviewed the proposed relief requests, deferred test justifications, and other relevant sections of the IST program against the requirements of Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code pursuant to paragraph 50.55a of Part 50 to Title 10 of the Code of Federal Regulations (10 CFR). The findings of our review are discussed in the attached safety evaluation (SE), and a summary of the results is provided in the following paragraph.

Two relief requests were denied and one relief request was granted on a provisional or interim basis. Deferred test justification ROJ-V-2 should have been included in the IST program submittal as a relief request from the Code test method requirements. However, since there was sufficient information to evaluate the justification, the relief was evaluated and granted on a provisional basis as discussed in Section 2.1 of the SE. In addition, your September 15, 1995, submittal did not address the staff's concerns stated in our SE dated June 13, 1994, regarding three relief requests which were granted on an interim basis during the second 10-year interval. These were granted again for 60 days on an interim basis. Finally, the information pertaining to the exclusion of the reactor core isolation cooling (RCIC) system from the IST program does not appear to be justified by the Hatch Final Safety Analysis Report (FSAR) and the Technical Specifications (TS). Therefore, based on the information contained in the above documents, the staff has concluded that the RCIC system has a safety function. Consequently, the appropriate RCIC components should be included in your IST program for Hatch Units 1 and 2.

During the course of this review, we have determined that certain requirements of Section XI of the ASME Boiler and Pressure Vessel Code, for which you have requested relief are (i) impractical to perform, (ii) compliance would result in hardship without a compensating increase in safety, or (iii) the proposed alternative testing, specified in your submittals, ensures an acceptable level of quality and safety. For these cases, relief from the Code requirements is granted as requested, with provisions, or on an interim basis, pursuant to 10 CFR 50.55a(a)(3)(i), 50.55a(a)(3)(ii), or 10 CFR 50.55a(f)(6)(i). The results of the NRC staff's evaluation of all relief requests are summarized in the enclosed SE.

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For the reliefs that are granted as requested or granted with provisions, the staff has determined that these reliefs are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest giving due consideration to the burden that could result if the requirements were imposed on your facility.

Based on our review, we have determined that the hatch IST program reflects compliance with Commission requirements of 10 CFR 50.55a(f) and with ASME Code, Section XI, except where relief has been granted, granted with provisions, or granted on an interim basis. Therefore, it is acceptable for implementation provided the items identified in the SE are addressed within the time frame specified therein.

Program changes such as additional relief requests or changes to relief requests should be submitted for staff review and should not be implemented prior to review and approval by the NRC; however, new or revised relief requests meeting the positions in Generic Letter 89-04, Enclosure 1, should be submitted to the NRC staff but can be implemented provided the guidance in Generic Letter 89-04, Section D, is followed. Program changes that involve additions or deletions of components from the IST program should be provided to the NRC.

We request that you provide a response to the specific issues outlined in our SE within 60 days from the date of this letter. This completes our actions on TAC Nos. M93072 and M93073. If you have any comments regarding this matter, please contact Kahtan N. Jabbour at (301) 415-1496.

This requirement affects nine or fewer respondents and, therefore, it is not subject to the Office of Management and Budget review under P.L. 96-511.

Sincerely,
Original signed by:

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosure: Safety Evaluation

cc w/encl: See next page

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April 12, 1996

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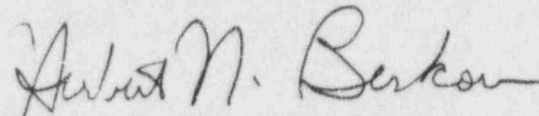
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Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
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Docket Nos. 50-321 and 50-366

Enclosure: Safety Evaluation

cc w/encl: See next page

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