

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

DECEMBER 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

December 1-31 The Unit started the reporting period at 60% power. At 0000 Hours on December 4, Reactor Power was raised to 80%. At 2300 hours on December 5, reactor power was lowered to 63% per the load dispatcher. At 0800 hours on December 7, reactor power was raised to 80%. At 0700 hours on December 19, reactor power was lowered to 66% for repairs to the MSIV limit switches. At 1500 hours on December 19, reactor power was reduced to 45% to reduce radiation exposure during MSIV limit switch repairs. At 2300 hours on December 19, Reactor power was raised to 65%. At 2300 hours on December 21, reactor power was raised to 80%. At 0700 hours on December 22, reactor power was raised to 94%. At 0115 hours on December 23, reactor power was reduced to 80% to adjust the flow control line. At 0700 hours on December 24, reactor power was raised to 90%. At 0400 hours on December 27, Reactor power was reduced to 80% due to the #2 and #4 control valve failure to fast close during a scheduled surveillance. At 0000 hours on December 28, reactor power was reduced to 60% for maintenance work on the #2 and #4 control valves. At 0000 hours on December 31, reactor power was raised to 90% power. The reactor was critical for 744 hours.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or technical specifications during the reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L26314	Main Steam Low pressure Isolation switches.	Install Pressure Snubbers	Could cause Unit trip due to spurious pressure pulsations.	Performed work per modification package #1-1-83-41.
L31235	RHR Min Flow Bypass Valves.	Modify bypass valve thermal overload bypass circuit.	May cause bypass valve to open on ECCS Actuation.	Revised wiring to conform to other ECCS Logic.
L31539	Aux. Elec. Equip. room air cooled condensing Fan.	Fan motor casing is cracked.	Redundant Ventilation Train still operable.	Replaced fan motor.
L43700	Diesel Generator air start motor.	Air Start Motors have 43 starts on them. Need to be replaced.	Per G. E. Recommendations, number of motor starts should not exceed 50.	Air start motors were changed out.
L43894	Drywell/Suppression Pool temperature recorder.	Recorder Sometimes Stops driving.	Still Provides temperature indication but cannot trend changes in temperature.	Changed pinion motor, adjusted clutch and drive belt.
L44008	Post Loca Monitor	Excessive Oscillations noted causing alarm for HI O ₂ to come up.	Rad Chem Technicians sampling drywell atmosphere to assure correct O ₂ & H ₂ concentration.	Found Slight zero shift at analyzer.
L44239	Core Plate DP Recorder.	Calibrate Core differential pressure recorder. Instrument Loop.	Core Plate DP close to exceeding Tech Spec. Limits.	Recalibrated per LIP-NB-05.
L44304	Drywell Equipment drain sump heat exchanger relief valve.	Relief Valve suspected of leaking.	Excessive heat inputs to RBEDT.	Bench tested relief valve at set pressure and verified no leaks.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L44378	Hydrogen Recomb- iner inlet flow loop.	Recalibrate flow loop.	Failed to meet required flow.	Completed LIS-HG-102 satisfactorily.
L44614	"B" Outboard MSIV	Limit switch out of ad- justment.	Failed to generate a half scram during the performance of LOS-RP-M1.	Reset limit switch to proper setpoint cycled valve 4 times and verified proper operation.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, December 1 through December 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-073-00	11/6/84	PCIS Group 1 Isolation Signal on Turbine Reset.
84-074-00	11/7/84	RWCU Isolation
84-075-00	11/11/84	1B RR Pump Suction Temperature RTD well leak
84-076-00	11/11/84	Unit 1 & 2 VR Isolation
84-077-00	11/15/84	Auto Start of B control room emergency makeup train.
84-078-00	11/17/84	Ammonia Chlorine ESF Actuation.
84-079-00	11/14/84	High Rad Door Open Without positive control.
84-080-00	11/20/84	Potential failure of safety related battery rack.
84-081-00	11/23/84	HI Suppression pool level HPCS suction swap.
84-082-00	11/24/84	RWCU Hi Δ Flow Isolation.
84-083-00	11/24/84	Unsecured Hi Rad Area.
84-084-00	11/30/84	Control Room HVAC Ammonia Detector Spurious Actuation.
84-085-00	11/29/84	Loss of feedwater heating procedure nonconservative with respect to analysis.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE January 10, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: December 1984 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY
 (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>744.0</u>	<u>6280</u>	<u>6280</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1165</u>	<u>1165</u>
7. HOURS GENERATOR ON LINE	<u>744.0</u>	<u>6055</u>	<u>6055</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1.0</u>	<u>1.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>2017106</u>	<u>16823289</u>	<u>16823289</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>664254</u>	<u>5470643</u>	<u>5470643</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>638963</u>	<u>5195062</u>	<u>5195062</u>
12. REACTOR SERVICE FACTOR	<u>100%</u>	<u>71.5%</u>	<u>71.5%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>84.8%</u>	<u>84.8%</u>
14. UNIT SERVICE FACTOR	<u>100%</u>	<u>68.9%</u>	<u>68.9%</u>
15. UNIT AVAILABILITY FACTOR	<u>100%</u>	<u>68.9%</u>	<u>68.9%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>82.9%</u>	<u>57.1%</u>	<u>57.1%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>79.7%</u>	<u>54.9%</u>	<u>54.9%</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0</u>	<u>15.1%</u>	<u>15.1%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A			

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: JANUARY 10, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761

MONTH: DECEMBER, 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____ 628 _____	17. _____ 966 _____
2. _____ 641 _____	18. _____ 998 _____
3. _____ 772 _____	19. _____ 678 _____
4. _____ 881 _____	20. _____ 742 _____
5. _____ 730 _____	21. _____ 930 _____
6. _____ 736 _____	22. _____ 1020 _____
7. _____ 869 _____	23. _____ 873 _____
8. _____ 879 _____	24. _____ 1004 _____
9. _____ 875 _____	25. _____ 1036 _____
10. _____ 862 _____	26. _____ 1006 _____
11. _____ 881 _____	27. _____ 861 _____
12. _____ 870 _____	28. _____ 613 _____
13. _____ 883 _____	29. _____ 661 _____
14. _____ 946 _____	30. _____ 873 _____
15. _____ 986 _____	31. _____ 979 _____
16. _____ 945 _____	

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle One
DATE January 10, 1985
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

REPORT MONTH DECEMBER 1984

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
21	84/12/19	F		0.0	A	5	Load drop for MSIV limit switch repairs.
22	84/12/28	F		0.0	A	5	Load drop for maintenance on #2 and #4 turbine control valves.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no safety relief valve operations during the reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-1298-84	1E12-C300A	Lube Pump Coupling
1-1304-84	1E12-F068A	Repack valve
1-1306-84	1E12-C003	Lube Pump Coupling
1-1338-84	1A D/G	Replace Air Start Motors
1-1339-84	1A D/G	Replace Lube Oil Filters
1-1340-84	1A D/G	Instrumentation Calibration.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes made to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

DECEMBER 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

December 1-18

The Unit started the reporting period at 95% power. At 1335 hours on December 14, a turbine trip and reactor scram occurred due to high turbine vibrations which was caused by a loose card in the turbine supervisory instrumentation cabinet. The reactor was critical for 325 hours and 35 minutes.

December 19 - 31

The reactor went critical at 1850 hours on December 19. At 0830 hours on December 20, the Main Generator was synchronized to the grid. At 1500 hours on December 20, Reactor power was raised to 27%. At 0700 hours on December 21, Reactor power was raised to 55%. At 2300 hours on December 24, reactor power was raised to 80%. At 2300 hours on December 27, reactor power was raised to 96%. The reactor was critical for 293 hours and 10 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

Amendment No. 6 - This amendment added a reactor scram on Low control rod drive pump discharge pressure as required by license condition 2.c(7).

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit C.1e during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L43660	Div 2 Battery Charger	Charger output voltage adjusting pot is set for max voltage but does not deliver adequate voltage.	Could eventually allow batteries to drain slowly.	Made adjustments to resistors to provide adequate voltage.
L43889	Containment vent & purge inlet valve.	Valve Failed LLRT	Potential Loss of Containment Integrity.	Replaced valve seat and reperformed LLRT.
L44078	Main Steam Tunnel Δ T recorder.	Alarm Setpoint is 28°F and indicated Δ T is 29°F	Recorder in constant alarm. Higher setpoint acceptable.	Raised setpoint to 32°F per temp. system change.
L44298	CY Tank Suction Valve to HPCS.	Valve will not open using control switch from control room.	Could lose CY supply to HPCS pump.	Found wire missing. Corrected per applicable wiring diagrams.
L44388	VR Exhaust Rad Monitor	Rad Monitor is reading upscale due to defective GM Tube.	Redundant monitors still operable.	Replaced defective GM Tube.
L44395	RHR full flow test valve.	Valve torques out before it is fully closed.	Valve can not fully close.	Readjusted torque switch setting.
L44501	Shutdown Cooling suction valve.	Valve is difficult to open.	Not able to run shutdown cooling through normal lineup.	Long term corrective action is currently being investigated.
L44505	Div II Post Loca O ₂ Monitor.	Recorder indicating \approx 12.5% O ₂ for several hours while deinerting drywell.	Rad Chem Technicians verified O ₂ concentration prior to entry.	Found poor solder connections on power supply.

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L44511	Inboard MSIV-LC depressurization valve	Valve would not fully open during surveillance LOS-MS-Q1.	Valve required to be operable per Tech. Spec. 3.6.1.4.	Repacked valve, adjusted limit switch and performed LLRT.
L44531	LPCI "B" testable check valve.	Valve will not cycle properly.	Proper operation observed with pump pressure present.	Replaced valve operator.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, December 1 through December 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-073-00	11/12/84	Reactor Water Cleanup Isolation on Δ T.
84-074-00	11/16/84	Reactor Water Cleanup Hi Area temperature isolation.
84-075-00	11/12/84	Loss of a RPS Bus.
84-076-00	11/20/84	Main Turbine Bypass Valve Inop.
84-077-00	11/15/84	2A Primary Containment Vacuum Breaker Cycling.
84-078-00	11/17/84	HPCS Suction Swap on the Hi Suppression pool level.
84-079-00	11/21/84	Reactor Water Cleanup Isolation on Hi Δ Flow.
84-080-00	11/21/84	Suppression Pool to Drywell Vacuum Breakers Open.
84-081-00	11/27/84	2A Primary Containment Vacuum Breaker Cycling.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE January 10, 1984
COMPLETED BY Randy

OPERATING STATUS

1. REPORTING PERIOD: December 1983
2. CURRENTLY AUTHORIZED POWER LEVEL (MWe-Net): 1036 DESIGN ELECTRIC POWER LEVEL
3. POWER LEVEL TO WHICH RESTRICTED
4. REASONS FOR RESTRICTION (IF ANY)

5. NUMBER OF HOURS REACTOR WAS CRITICAL
6. REACTOR RESERVE SHUTDOWN HOURS
7. HOURS GENERATOR ON LINE
8. UNIT RESERVE SHUTDOWN HOURS
9. GROSS THERMAL ENERGY GENERATED (MWh)
10. GROSS ELEC. ENERGY GENERATED (MWh)
11. NET ELEC. ENERGY GENERATED (MWh)
12. REACTOR SERVICE FACTOR
13. REACTOR AVAILABILITY FACTOR
14. UNIT SERVICE FACTOR
15. UNIT AVAILABILITY FACTOR
16. UNIT CAPACITY FACTOR (USING MDC)
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)
18. UNIT FORCED OUTAGE RATE
19. SHUTDOWNS SCHEDULED OVER NEXT 6 WEEKS
There is an outage scheduled to start on 1/15/84 and surveillances. This outage will last 6 weeks.
20. IF SHUT DOWN AT END OF REPORT PERIOD

D. DATA TABULATIONS

The following data tabulations are presented

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE January 10, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: December 1984 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>618.8</u>	<u>1611.8</u>	<u>1611.8</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>125.3</u>	<u>125.3</u>	<u>125.3</u>
7. HOURS GENERATOR ON LINE	<u>605.1</u>	<u>1537.4</u>	<u>1537.4</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>1781736</u>	<u>4512592</u>	<u>4512592</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>586372</u>	<u>1484995</u>	<u>1484995</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>562402</u>	<u>1392117</u>	<u>1392117</u>
12. REACTOR SERVICE FACTOR	<u>83.2%</u>	<u>90.8%</u>	<u>90.8%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>97.8%</u>	<u>97.8%</u>
14. UNIT SERVICE FACTOR	<u>81.3%</u>	<u>86.6%</u>	<u>86.6%</u>
15. UNIT AVAILABILITY FACTOR	<u>81.3%</u>	<u>86.6%</u>	<u>86.6%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>73.0%</u>	<u>75.7%</u>	<u>75.7%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>70.1%</u>	<u>72.7%</u>	<u>72.7%</u>
18. UNIT FORCED OUTAGE RATE	<u>18.7%</u>	<u>13.5%</u>	<u>13.5%</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 There is an outage scheduled to begin on March 1, 1985 for maintenance and surveillances. This outage is expected to last approximately nine weeks.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: January 10, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761
MONTH: December 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>1030</u>
2.	<u>1025</u>
3.	<u>1032</u>
4.	<u>1041</u>
5.	<u>1039</u>
6.	<u>1037</u>
7.	<u>1042</u>
8.	<u>1043</u>
9.	<u>1041</u>
10.	<u>1044</u>
11.	<u>1049</u>
12.	<u>1049</u>
13.	<u>1044</u>
14.	<u>587</u>
15.	<u>0</u>
16.	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>0</u>
18.	<u>0</u>
19.	<u>0</u>
20.	<u>176</u>
21.	<u>538</u>
22.	<u>714</u>
23.	<u>775</u>
24.	<u>865</u>
25.	<u>857</u>
26.	<u>930</u>
27.	<u>968</u>
28.	<u>901</u>
29.	<u>823</u>
30.	<u>889</u>
31.	<u>980</u>

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH DECEMBER 1984

DOCKET NO. 050-374

UNIT NAME LaSalle Two

DATE JANUARY 10, 1984

COMPLETED BY Randy S. Dus

TELEPHONE (815)357-6761

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
5	84/12/14	F		138.9	A	3	Turbine trip due to a loose card in the turbine supervisory panel.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
12/14/84	2B21-F013C	1 Automatic	1086 PSIG	RX Scram
12/14/84	2B21-F013D	1 Automatic	1086 PSIG	RX Scram.
12/14/84	2B21-F013E	1 Automatic	1086 PSIG	RX Scram
12/14/84	2B21-F013K	1 Automatic	1086 PSIG	RX Scram.
12/14/84	2B21-F013S	1 Automatic	1086 PSIG	RX Scram
12/14/84	2B21-F013U	1 Automatic	1086 PSIG	RX Scram.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-1293-84	2E22-C002	Lube Pump Coupling
2-1297-84	2B D/G	Change Oil

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes made to the radioactive waste treatment system during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

January 7, 1985

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering December 1 through December 31, 1984.

Very truly yours,

G. J. Diederich
G. J. Diederich 1/7/85
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. [unclear], CECo
[unclear] Farrar, CECo
D. O. Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident
J. M. Nowicki, Asst. Comptroller

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