NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
H. B. Robinson Plant, Unit No. 2	0 5 0 0 0 2 6 1	1 OF 0 2

TITLE (4)

Reactor Trip Due to High Level in "A" Steam Generator

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 1	0 9	8 5	8 5	— 0 0 5	— 0 0	0 2	0 8	8 5		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
		20.402(b)		20.405(c)	X	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	0 1 4	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Carson L. Wright	8 0 3 — 3 8 3 — 4 5 2 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO		0 4	0 5	8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 9, 1985, the reactor was critical, and the turbine was on line increasing load. At 0402 hours, a reactor trip occurred at 14% power due to a turbine trip from a high level in "A" steam generator. It appears that erratic operation of "A" steam generator level control circuitry at low power levels caused "A" feedwater egulating valve to open further than desired. This caused "A" steam generator level to exceed the high level setpoint which tripped the turbine and thus the reactor.

The cause of the erratic operation of "A" steam generator level control at low power levels has not been identified. Troubleshooting will continue should erratic operation recur during subsequent startups. A supplemental report will be filed when a cause is found and corrective actions are determined.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
H. B. Robinson Plant, Unit No. 2	0 5 0 0 0 2 6 1	8 5	— 0 0 5	— 0 0	0 2 OF 0 2

TEXT *(If more space is required, use additional NRC Form 366A's)* (17)

On January 9, 1985, the turbine was being loaded. At 0402 hours, a reactor trip occurred at 14% power due to a turbine trip from a high level in "A" steam generator. The high level signal also trips the feed pumps. The reactor trips if the turbine trips above 10% power.

During the recent Plant startups, the three steam generators experienced level control problems. The level control systems were inspected mechanically and electrically for potential problems. Maintenance has corrected the problems in the level controllers for "B" and "C" steam generator. A failed capacitor contributed to electronic circuit noise in "B" level controller. Debris in the instrument air side of "C" valve controller caused increased response time. Investigation of the "A" steam generator level control circuitry, manual and automatic, did not reveal any failures or abnormal signal conditions; therefore, a problem has yet to be found. The erratic operation of "A" steam generator level control circuitry caused "A" feedwater regulating valve to open further than desired. This overfeeding of "A" steam generator gave a high level in the steam generator which tripped the turbine and thus the reactor.

Troubleshooting will continue on the "A" steam generator level control circuitry to identify and correct the cause of the erratic operation at low power levels during subsequent startups. A supplemental report will be filed when a cause is found and corrective actions are determined.

CP&L
Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 85-005

Dear Sir:

In accordance with 10CFR50.73, Licensee Event Report, the enclosed Licensee Event Report is submitted. This report fulfills the requirements for a written report within (30) days of a reportable event and is in accordance with the format set forth in NUREG-1022, September, 1983.

Very truly yours,

R. E. Morgan
General Manager
H. B. Robinson S. E. Plant

CLW/ac

Enclosure

cc: INPO
H. E. P. Krug
J. N. Grace

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