



PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Red Wing, Minnesota

UNITS 1 AND 2



INSERVICE INSPECTION - EXAMINATION SUMMARY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT - UNIT I
OCTOBER 23 to NOVEMBER 4, 1984
STEAM GENERATOR TUBE - EDDY CURRENT EXAMINATION
INSPECTION PERIOD 3

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PDR

NORTHERN STATES POWER COMPANY
MINNEAPOLIS, MINNESOTA

REPORT DATE: January 8, 1985

COMMERCIAL SERVICE
DATE: DECEMBER 16, 1973



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INSERVICE INSPECTION - EXAMINATION SUMMARY

FOR THE

PRAIRIE ISLAND NUCLEAR GENERATING PLANT - UNIT I

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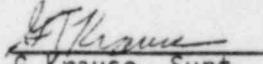
STEAM GENERATOR TUBE - EDDY CURRENT EXAMINATION

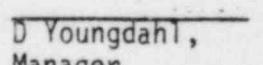
Commerical Service
Date: 12-16-73

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NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT - UNIT I
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INSERVICE INSPECTION EXAMINATION SUMMARY
FOR THE
PRAIRIE ISLAND NUCLEAR GENERATING PLANT
UNIT I
OCTOBER 23 to NOVEMBER 4, 1984

1.0 Introduction

This report is a summary of the steam generator tube eddy current examinations performed on steam generators No. 11 and 12 at Prairie Island-Unit I. The examinations were performed during the period from October 23 through November 4, 1984 after tube leakage was detected via secondary side chemical analysis. Prairie Island-Unit I began commercial operation on December 16, 1973.

2.0 Summary

On October 21, 1984, Prairie Island Unit 1 began shutting down due to primary-to-secondary steam generator tube leakage of approximately 0.33 gpm which escalated to approximately 0.65 gpm before hot shut-down conditions were achieved. (Technical Specifications of 1.0 gpm were not exceeded.) Westinghouse was mobilized to provide necessary eddy current examinations and tube plugging services. What follows is a synopsis of the relevant events which occurred during the brief outage period defined previously.

Primary man-ways were removed on October 23, 1984, and visual inspections of both hot and cold leg tubesheets were performed in steam generators #11 and #12 with secondary side hydrostatic pressures exceeding 600 psi. A small leak (approximately one drop per second) was observed in the cold leg of steam generator #11 (row 25, column 87). No visible leakage was observed in steam generator #12 at this time.

Multifrequency eddy current examinations were performed in steam generator #11 on row 25, column 87, and approximately 25 surrounding tubes. These examinations were conducted for the full length of each tube with standard Zetec MIZ-12 Data Acquisition equipment. The analysis of this data showed that the defect in tube #R25 C87 was located at the seventh tube support plate, cold leg. Analysis of the data for the surrounding tubes indicated that, while some tube degradation at locations other than the seventh tube support plate were observed, no other defective tubing in this sample was evident.

On October 24, 1984, a second tubesheet visual inspection was conducted on steam generator #12 while under secondary atmospheric head pressure only and an extremely small leak (approximately one drop every three minutes) was observed in the hot leg, row 17, column 42. An eddy current examination was performed on this tube and the defective area was determined to be in the hot leg tubesheet region (beginning at the top of the roll transition extending to approximately 15 inches above the tube end). This defective area exhibited multiple, discrete tube wall penetrations exceeding the technical specifications plugging limit.

The following tubes were mechanically plugged based on visual leakage and/or eddy current examinations:

<u>S/G-#11</u>	<u>Location of Defect</u>
R25 C87	#7 Tube Support Plate - Cold Leg
R7 C94	Tubesheet - Cold Leg (Previous Commitment)*
<u>S/G-#12</u>	<u>Location of Defect</u>
R17 C42	Tubesheet - Hot Leg
R13 C15	Tubesheet - Hot Leg (Previous Commitment)*
R6 C93	Tubesheet - Cold Leg (Previous Commitment)*

*Note: See Attachment #1 for details of this commitment.

After Unit 1 returned to power, on October 28, 1984, another tube leak was detected by plant chemistry personnel in steam generator #12. Before the plant could get shut-down this leak had grown to approximately 3.5 gpm. Steam generator #11 exhibited minimum leakage per plant chemistry, therefore a Technical Specification, C-1 type sampling program was devised for steam generator #12 only. This sample included all tubes with previously reported tube wall degradations (20%), plus approximately 300 tubes in the region of the tube bundle where tube wall degradations were historically prevalent. Westinghouse performed the ET inspection of this sample which consisted of full length tube examinations utilizing standard Zetec MIZ-12 Data Acquisition equipment. The data analysis was performed by Conam Inspection using Zetec digital analysis (DDA-4) equipment.

On October 31, 1984, a visual inspection of the hot and cold leg tubesheets were performed on S/G #12 and a relatively large leak was evident (small continuous stream) in the hot leg, row 18, column 45. This tube was eddy current examined and the results exhibited a defective area of approximately ten inches in length, starting at about six inches above the tube end and extending to about sixteen inches above the tube end, hot leg (tubesheet crevice region). Again, this area consisted of multiple, large tube wall penetrations with the largest observed at approximately ten inches above the tube end.

When this information became available, NSP initiated an inspection which consisted of the hot leg tubesheet region up to the first tube support plate for approximately 100% of all S/G #12 tubing (26 tubes were not inspected; these tubes were inaccessible due to positioning fixture and template plugs). These examinations were performed in addition to the C-1 sampling which was previously discussed. Also, a special larger diameter (0.740 inch) probe was used in the tubesheet region examinations which improved sensitivity by minimizing probe wobble. The tubesheet region inspection resulted in the detection of one (1) additional tube (row 9, column 26) which exhibited an indication located just above the roll transition area (approximately 4.8 inches from tube end), hot leg. Although this indication was of a very small amplitude (1.36 volts), its phase suggested it to be in excess of the plugging limit (50% tube wall penetration) and was consequently plugged.

The following tubes in S/G #12 were mechanically plugged based on visual leakage, eddy current examinations, or for preventative measures:

<u>Tube Number</u>	<u>Defect Location/Severity</u>	<u>Comments</u>
R18 C45	Tubesheet - Hot Leg/100%	Observed Leak Visually
R9 C26	Tubesheet - Hot Leg/92%	
R37 C75	First Tube Support Plate - Cold Leg/46%	Preventative
R34 C78	First Tube Support Plate - Cold Leg/42%	Preventative

Prairie Island - Unit I is scheduled for its ninth refueling outage to tentatively begin on January 5, 1985. During this time NSP will conduct eddy current examinations on 100% of the tubing in steam generators No. 11 & 12. In addition to full-length tube examinations, an extensive investigation of the tubing in the hot-leg (inlet), tubesheet crevice region for both generators shall be performed.

3.0 Examination Method

As previously stated, Westinghouse was contracted to perform the eddy current examinations utilizing a Zetec MIZ-12 data acquisition system. This system provides multi-frequency capabilities for use in determining tube integrity. The frequencies utilized for each examination were 400 KHz and 100 KHz in the differential mode, with 210 KHz and 100 KHz in the absolute mode.

Analysts from Zetec and Conam were utilized to evaluate the examination data. All data was subjected to 100% re-evaluation (2nd level review) before the final results were tabulated. Analysis was performed using digital evaluation equipment (DDA-4) which provides an increase in analytical capability.

4.0 Equipment and Materials

All equipment used in the examinations are listed by either serial number or type, as applicable, along with their respective calibration dates in Table III of Appendix A.

5.0 Personnel

Northern States Power Company contracted Westinghouse to perform the examinations, with Zetec and Conam providing technical/analytical support. Hartford Steam Boiler Inspection and Insurance Company, representing ANI, provided the Authorized Inspection.

All personnel involved in the performance or evaluation of examinations are listed, along with their title, organization and ASNT certification level, in Table I of Appendix A. Certification for examination personnel are maintained on file by Northern States Power Company.

6.0 Examination Results

A summary of the tubes exhibiting eddy current indications is included in the cumulative listing of indications found in Appendices B and C for steam generators #11 and #12, respectively. The total numbers of tubes plugged to date are also exhibited in these appendices.

APPENDIX A

**TABLE I - PERSONNEL
TABLE II - PROCEDURE LISTING
TABLE III - EQUIPMENT AND MATERIALS**

PERSONNEL LISTING

EXAMINER	TITLE	ORGANIZATION	ASNT	LEVEL				
			UT	PT	MT	VT	ET	RT
BEHARY, G.T.	TECHNICIAN	W ⁽¹⁾					I	
BURGESS, W.A.	TECHNICIAN	W					II	
JENKINS, G.P.	TECHNICIAN	W					II	
POLLINE, R.A.	COORDINATOR	W					II	
PFARR, T.A.	TECHNICIAN	W					II	
BRONSON, J.I.	TECHNICIAN	W					I	
HAZEN, R.P.	TECHNICIAN	W					II	
HOSELY, R.	COORDINATOR	W					I	
ROSSI, W.	TECHNICIAN	W					I	
TOMMARELLO, D.	TECHNICIAN	W					II	
GILBERT, B.F.	EVALUATOR	ZETEC ⁽²⁾					IIA	
SIEGEL, J.D.	EVALUATOR	ZETEC					IIA	
CHAMBERS, D.M.	EVALUATOR	CONAM ⁽³⁾					IIA	
LOFGREN, R.	EVALUATOR	CONAM					IIA	
MARLOW, R.E.	EVALUATOR	CONAM					III	
ANDERSON, M.T.	M&SP ENGINEER	NSP						
DAHLMAN, L.C.	M&SP SPECIALIST	NSP						
HUGHES, R.	ANII	HARTFORD STEAM BOILER INSPEC- TION AND INSURANCE CO.						
FOOTNOTES:	(1) ORGANIZATION	(1) Westinghouse Electric Corporation Nuclear Services Division P.O. Box 2728 Pittsburgh, PA 15230						
		(2) ZETEC P.O. Box 140 Issaquah, WA 98027						
		(3) Conam Inspection 660 South 31st Street Richmond, CA 94804						

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND UNIT II

PROCEDURE LISTING

APPENDIX A
TABLE II
PAGE 1 of 1

PROCEDURE NUMBER AND REVISION	FIELD CHANGE	PROCEDURE TITLE	PLANT APPROVAL DATE	FIELD CHANGE REMARKS	CHANGE DESCRIPTION
MRS 2.4.2 Gen - 23, Rev. 5 with Generic Change #1	NONE	Multi-Frequency Eddy Current Inspection of Steam Generator Tubing - Preservice and Inservice	10-23-84	NONE	

MTA112884WMH01

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EQUIPMENT AND MATERIALS

APPENDIX A
TABLE III
PAGE 1 OF 2

MATERIAL OR EQUIPMENT		TYPE OR SERIAL NUMBER	CALIBRATION DATE OR BATCH NUMBER	REMARKS
Brush Recorder(s)	S/N	0533 01307 01308 0644 0025 30365	8-3-84 9-10-84 9-12-84 8-2-84 7-5-84 9-11-84	
Tape Recorder(s) HP 3968A2	S/N	2105A01653 2105A01491 00947 00946 0681 30230 0999 30246	9-24-84 7-11-84 6-8-84 7-30-84 8-3-84 7-27-84 9-17-84 6-8-84	
MIZ-12 Main Frame	S/N	30250 0682 0557	10-21-84 5-29-84 9-27-84	
Freq. Plug-In	S/N	0561 0684 0552 0558 30247 0636 00890 0584 0575 0568 0638 0709 0923 00877	6-7-84 6-1-84 6-7-84 6-4-84 10-22-84 6-7-84 10-20-84 6-4-84 9-27-84 9-27-84 6-26-84 9-28-84 8-9-84 6-19-84	
MIZ-12 Mixer Plug-In	S/N	0579 0640 0563 0571 0666 01627 0562 00881 0597 0598	6-7-84 6-4-84 6-7-84 6-4-84 9-26-84 9-28-84 6-26-84 6-21-84 9-19-84 6-8-84	

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EQUIPMENT AND MATERIALS

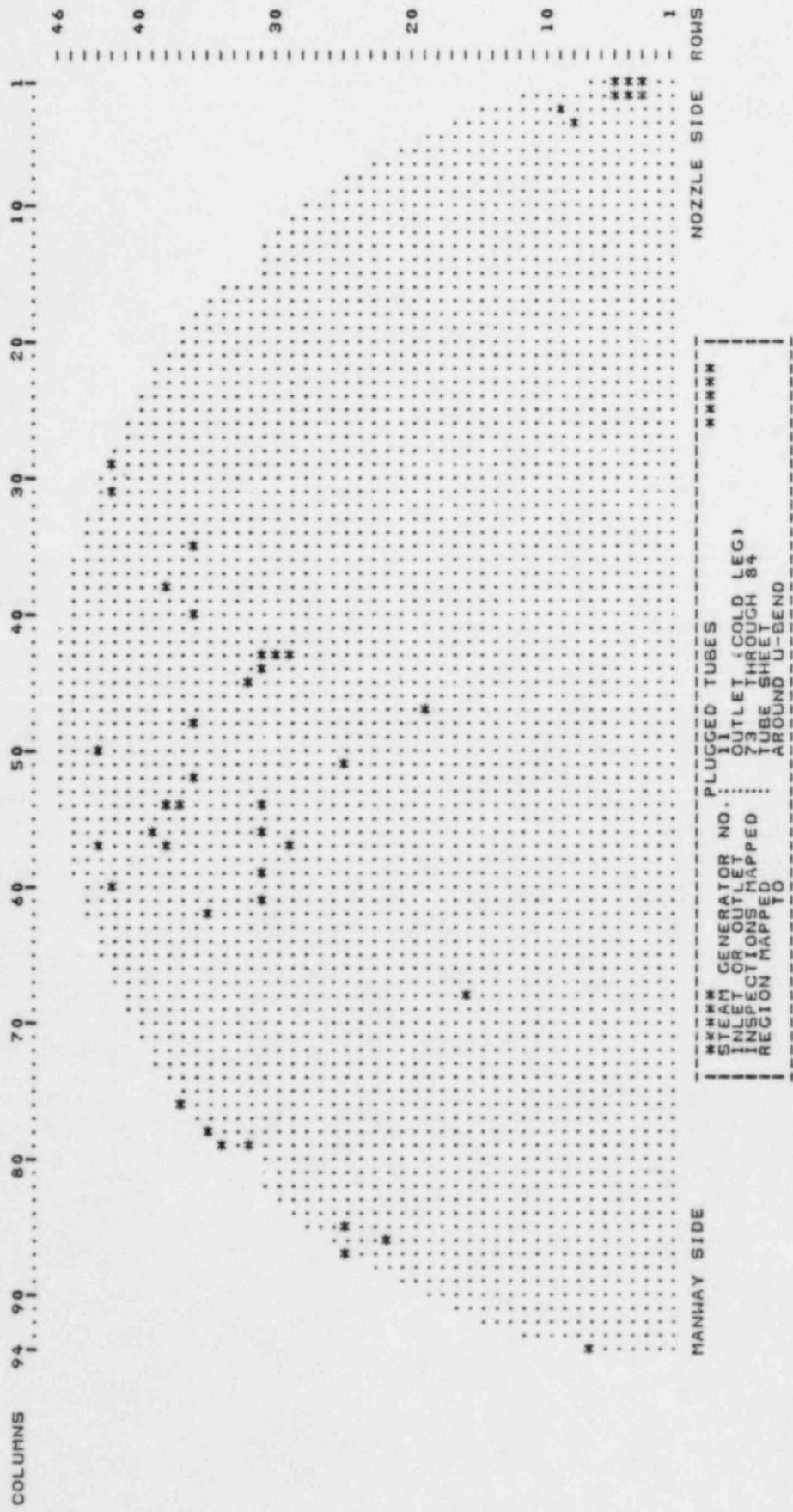
APPENDIX A
 TABLE III
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MATERIAL OR EQUIPMENT		TYPE OR SERIAL NUMBER	CALIBRATION DATE OR BATCH NUMBER	REMARKS
MIZ-12 Display Module/Scope	S/N	0266 0006 0698 0151 0537 0268 00923 30240 00918 0556	8-6-84 6-11-84 10-21-84 6-14-84 9-11-84 7-13-84 7-13-84 8-7-84 8-6-84 9-25-84	
Vector Analyzer	S/N	00942 00938	7-3-84 9-19-84	
Calibration Standards Inconel 600	WEM Z - Z - WEM Z - Z -	02660 1411 1341 02680 1399 1429	AVB Standard Defect Standard ABS Standard AVB Standard ABS Standard Defect Standard	

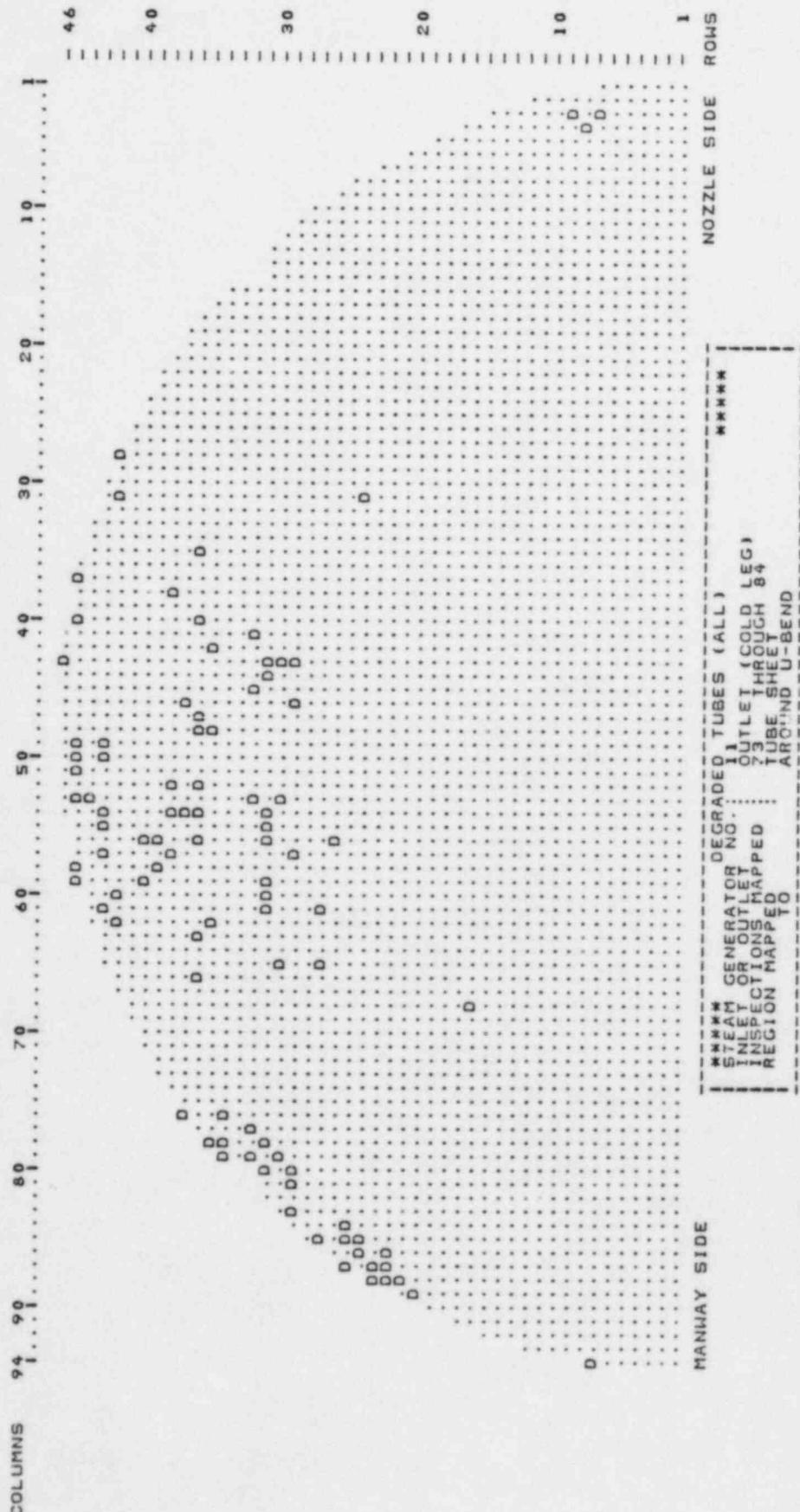
APPENDIX B

**STEAM GENERATOR NO. 11
EDDY CURRENT EXAMINATION RESULTS AND TUBE
SHEET MAPS**

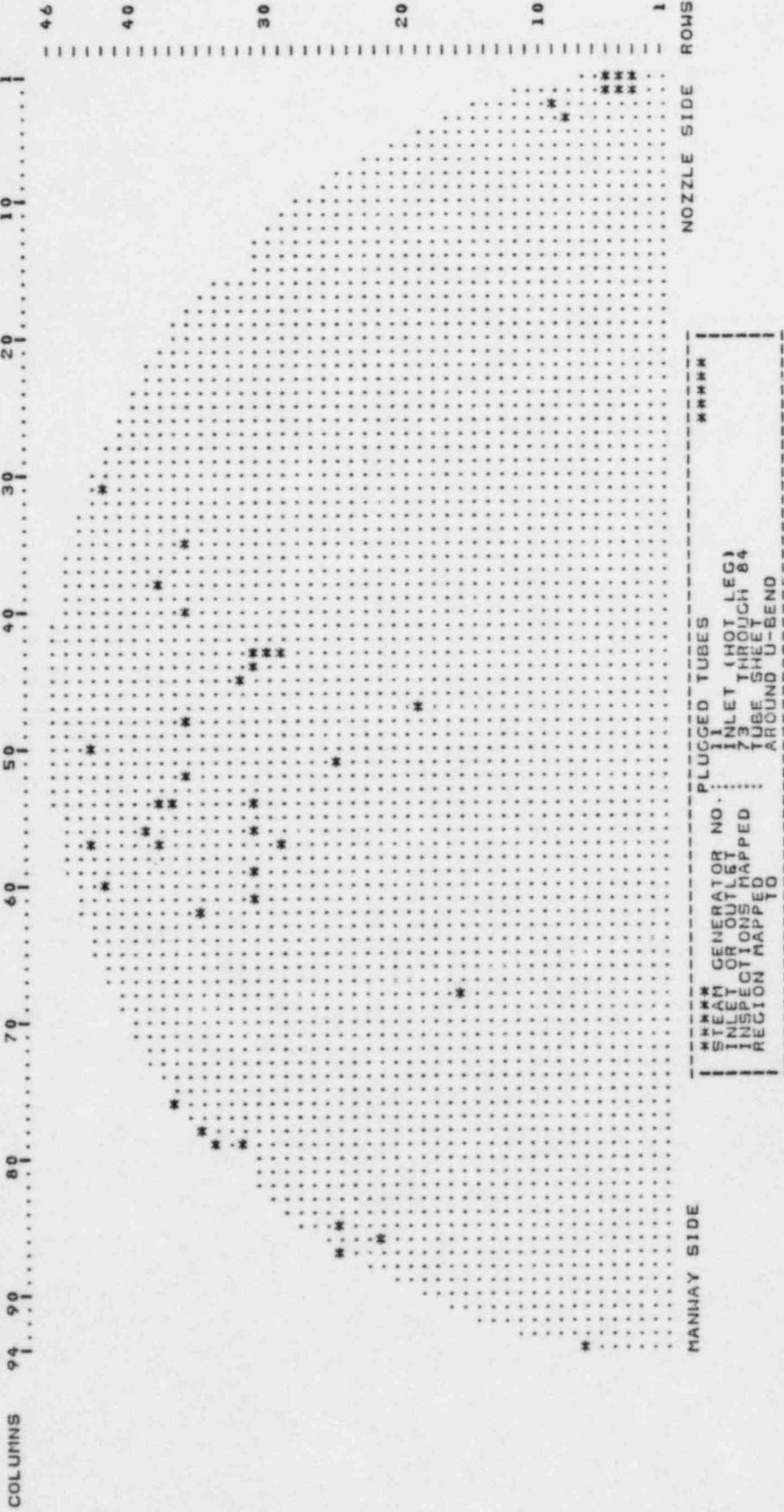
PRAIRIE ISLAND POWER PRODUCTION MATERIALS AND SPECIAL PROCESSES SECTION
NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT STEAM GENERATOR TUBE MAP - WESTINGHOUSE SERIES 51



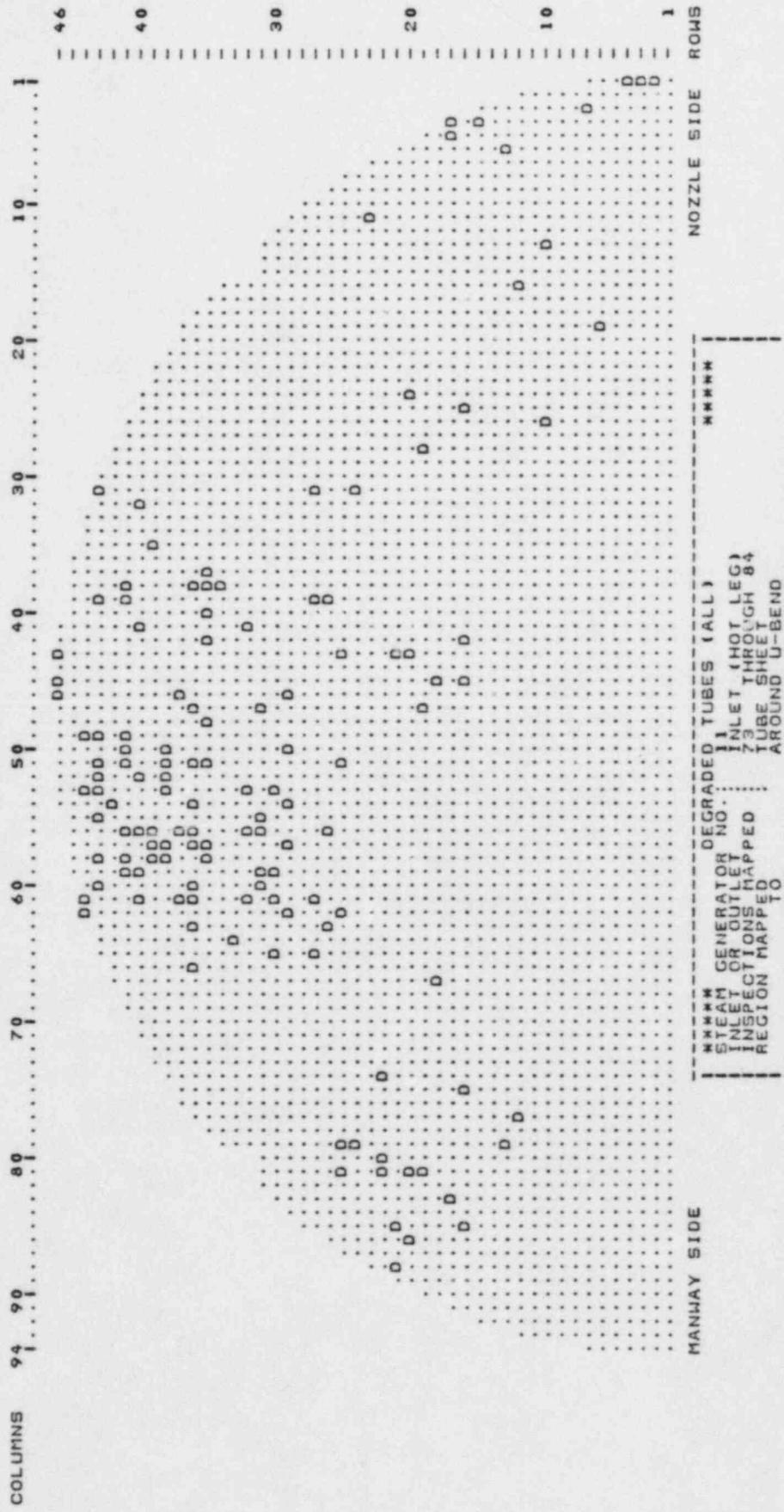
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NUCLEAR GENERATING PLANT STEAM GENERATOR TUBE MAP - WESTINGHOUSE SERIES 51



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POWER PRODUCTION MATERIALS AND SPECIAL PROCESSES SECTION
PRAIRIE ISLAND NUCLEAR GENERATING PLANT STEAM GENERATOR TUBE MAP - WESTINGHOUSE SERIES 51



DEC 31, 1984

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

DEC 31, 1984

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF PLUGGED STEAM GENERATOR TUBES

GEN NO.	SIDE	ROW	COL	YEAR	REMARKS/COMMENTS
11	OUTLET	36	48	8	A/VB
			52	8	A/VB
			54	8	A/VB
			56	8	S/I
			58	8	A/VB
			60	8	A/VB
			64	8	S/I
			67	8	S/I
			71	8	S/I
			76	8	S/I
			80	8	A/VB
			87	8	A/VB

DEC 31, 1984

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
11	INLET	2	1	79-	5 " ABOVE TUBE SH		THINNING	</>0	
				63-	58 " ABOVE TUBE SH		THINNING		
3		1		79-	55 " ABOVE TUBE SH		THINNING		
4		1	9	79-	35 " ABOVE TUBE SH	1ST SUP	THINNING		
6		1	9	83-	5 " ABOVE TUBE SH	1ST SUP	THINNING		
10		1	9	83-		1ST SUP	THINNING		
				126		2ND SUP	THINNING		
12		7	7	82-		1ST SUP	THINNING		
13		6		82-		1ST SUP	THINNING		
				79		1ST SUP	THINNING		
15		7	4	83-	1 " ABOVE TUBE SH		THINNING		
16		2	4	83-		4TH SUP	THINNING		
				45		4TH SUP	THINNING		
				755		4TH SUP	THINNING		
				8		2ND SUP	THINNING		
17		4		83-	1 " ABOVE TUBE SH	1ST SUP	THINNING		
				5	1 " ABOVE TUBE SH	1ST SUP	2 " ABOVE		
				8		1ST SUP	THINNING		
18		4		83-		1ST SUP	THINNING		
19		4		83-	25 " ABOVE TUBE SH	1ST SUP	THINNING		
20		4		83-	4 " ABOVE TUBE SH	1ST SUP	THINNING		
21		4		83-	4 " ABOVE TUBE SH	2ND SUP	THINNING		
22		4		84-		7TH SUP	THINNING		
23		4		84-		1ST SUP	THINNING		
24		4		84-		1ST SUP	THINNING		
25		4		84-	19" BELOW TUBE SH	1ST SUP	THINNING		
				56		1ST SUP	THINNING		
26		3	9	83-		2ND SUP	THINNING		
				63		2ND AVB	THINNING		
27		3	1	83-		3RD AVB	THINNING		
				39		3RD AVB	THINNING		
				61		4TH AVB	THINNING		
				65		4TH AVB	THINNING		
29		4	6	83-		1ST SUP	THINNING		
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						1ST AVB	THINNING		

DEC 31, 1984

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

DEC 31, 1984

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
11	INLET	36	47	83-	4TH AVB		THINNING		
			51	-	3RD AVB		THINNING		
			54	-	3RD AVB		THINNING		
			56	-	2ND AVB		THINNING		
			57	-	2ND AVB		THINNING		
			60	-	2ND AVB		THINNING		
			61	-	2ND AVB		THINNING		
			63	-	2ND AVB		THINNING		
			66	-	2ND AVB		THINNING		
37		46		-	2ND AVB		THINNING		
		56		-	2ND AVB		THINNING		
		61		-	2ND AVB		THINNING		
38		50		-	2ND AVB		THINNING		
		52		-	2ND AVB		THINNING		
		53		-	2ND AVB		THINNING		
		57		-	2ND AVB		THINNING		
		58		-	2ND AVB		THINNING		
39		55		-	2ND AVB		THINNING		
		56		-	2ND AVB		THINNING		
		57		-	2ND AVB		THINNING		
		58		-	2ND AVB		THINNING		
40		32		-	2ND AVB		THINNING		
		41		-	2ND AVB		THINNING		
		52		-	2ND AVB		THINNING		
		56		-	2ND AVB		THINNING		
		59		-	2ND AVB		THINNING		
		61		-	2ND AVB		THINNING		

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PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	X	REMARKS/COMMENTS
11	INLET	40	61	83-	RD AVB		THINNING	4	
		41	38		RD AVB		THINNING	220	
				79	RD AVB		THINNING	220	
				49	RD AVB		THINNING	220	
				50	RD AVB		THINNING	220	
				51	RD AVB		THINNING	220	
				56	RD AVB		THINNING	220	
				58	RD AVB		THINNING	220	
				59	RD AVB		THINNING	220	
		42	54		RD AVB		THINNING	220	
		43	31		RD AVB		THINNING	220	
			39		RD SUP		THINNING	220	
				49	RD AVB		THINNING	220	
				51	RD AVB		THINNING	220	
				52	RD AVB		THINNING	220	
				53	RD AVB		THINNING	220	
				55	RD AVB		THINNING	220	
				56	TUBE SH		THINNING	220	
				58	2ND AVB		THINNING	220	
		44	56		4TH AVB		THINNING	220	
			59		1ST SUP		THINNING	220	
			61		1ST SUP		THINNING	220	
			62		1RD AVB		THINNING	220	
		46	44		2ND AVB		THINNING	220	
	OUTLET	7	94		1 " ABOVE TUBE		THINNING	220	
			43		ABOVE TUBE		THINNING	220	
			68		ABOVE TUBE		THINNING	220	
		16	69		ABOVE TUBE		THINNING	220	
		22	88		ABOVE TUBE		THINNING	220	
			86		ABOVE TUBE		THINNING	220	
			87		ABOVE TUBE		THINNING	220	
		23	87		ABOVE TUBE		THINNING	220	
			88		ABOVE TUBE		THINNING	220	
		24	91		ABOVE TUBE		THINNING	220	
			85		ABOVE TUBE		THINNING	220	
			66		ABOVE TUBE		THINNING	220	
		25	84		ABOVE TUBE		THINNING	220	
			85		AVB		THINNING	220	

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PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
11	OUTLET	25	85	83-	1ST SUP		THINNING	43	
			87	84-	7TH SUP		THINNING		
		26	56	81-	4TH AVB		THINNING		
		27	61	81-	1ST AVB		THINNING		
			65	81-	3RD AVB		THINNING		
			85	83-	1ST SUP		THINNING		
			84-	84-	1ST SUP		THINNING		
		29	43	81-	2ND AVB		THINNING		
			46	81-	1ST AVB		THINNING		
				57	2ND AVB		THINNING		
				80	3RD AVB		THINNING		
				81	1ST SUP		THINNING		
				83	2ND SUP		THINNING		
		30	43	81-	1ST AVB		THINNING		
				51	2ND AVB		THINNING		
				57	3RD AVB		THINNING		
				80	4TH AVB		THINNING		
				81	1ST SUP		THINNING		
				83	2ND SUP		THINNING		
				30	1ST AVB		THINNING		
				51	2ND AVB		THINNING		
				53	3RD AVB		THINNING		
				65	4TH AVB		THINNING		
				79	1ST SUP		THINNING		
		31	43	81-	2ND AVB		THINNING		
				51	3RD AVB		THINNING		
				53	4TH AVB		THINNING		
				65	1ST AVB		THINNING		
				79	2ND AVB		THINNING		
				44	3RD AVB		THINNING		
				54	4TH AVB		THINNING		
				55	1ST SUP		THINNING		
				56	2ND AVB		THINNING		
				59	3RD AVB		THINNING		
				60	4TH AVB		THINNING		
				61	1ST AVB		THINNING		
				78	2ND AVB		THINNING		
		32	41	81-	3RD AVB		THINNING		
				45	4TH AVB		THINNING		
				53	1ST SUP		THINNING		
				77	2ND SUP		THINNING		
				79	1ST SUP		THINNING		
				34	2ND SUP		THINNING		
				76	1ST SUP		THINNING		
				78	2ND SUP		THINNING		
				79	1ST AVB		THINNING		
		35	42	81-	2ND AVB		THINNING		
				48	4TH AVB		THINNING		
				62	4TH AVB		THINNING		
				78	2ND SUP		THINNING		
				36	35				
				35	1ST AVB		THINNING		
				40	2ND AVB		THINNING		
				47	3RD AVB		THINNING		
				48	4TH AVB		THINNING		
				52	1ST AVB		THINNING		
				46	2ND AVB		THINNING		

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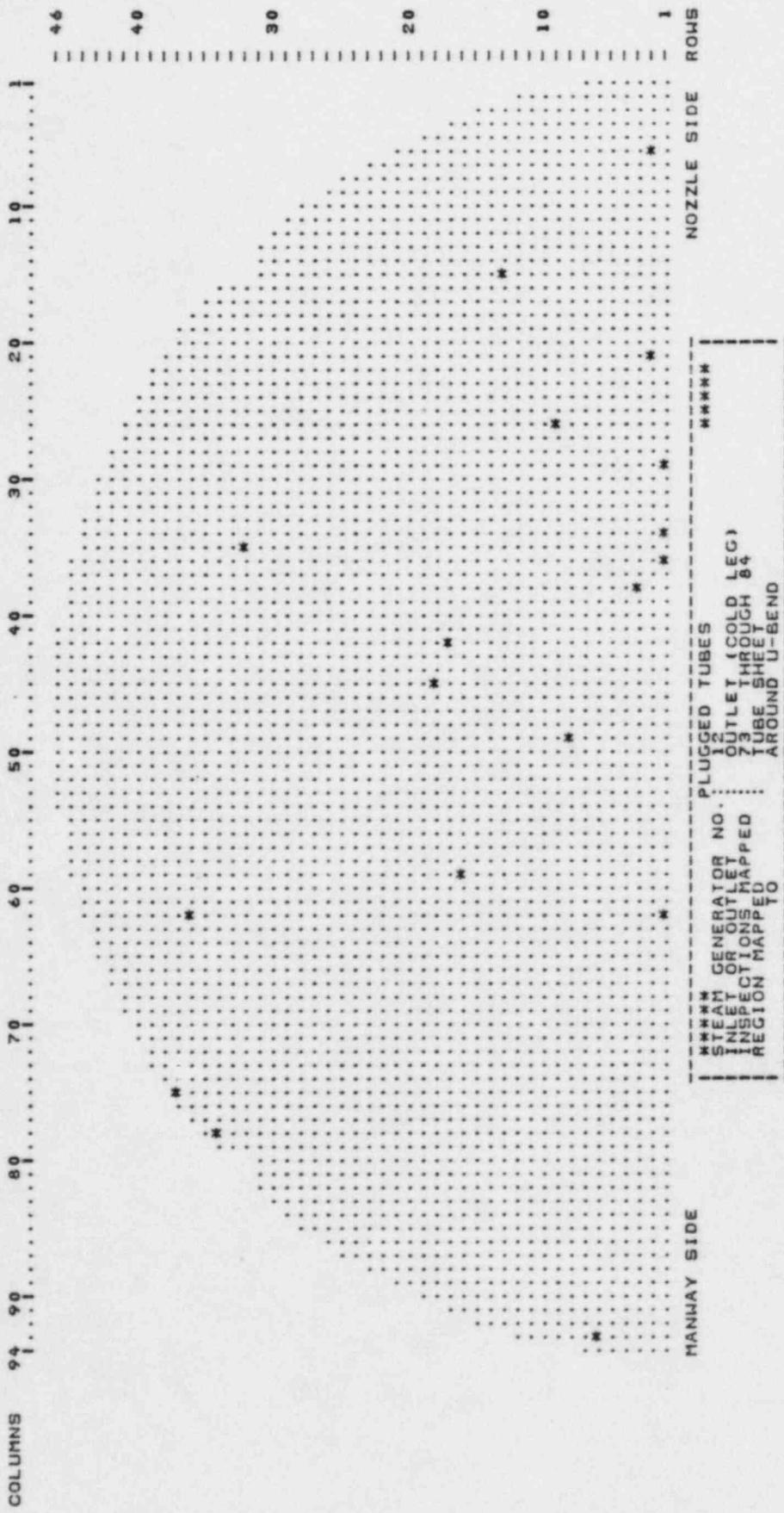
PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
11	OUTLET	36	56	81-	4 TH AVB		THINNING	0	
			63	-	3 RD AVB		THINNING	0	
			66	-	3 RD AVB		THINNING	0	
		37	46	-	3 RD AVB		THINNING	0	
			54	-	3 RD AVB		THINNING	0	
			57	-	1 ST SUP		THINNING	0	
		38	57	-	4 TH AVB		THINNING	0	
			56	-	3 RD AVB		THINNING	0	
			58	-	2 ND AVB		THINNING	0	
			59	-	3 RD AVB		THINNING	0	
		40	60	-	4 TH AVB		THINNING	0	
			60	-	3 RD AVB		THINNING	0	
			62	-	3 RD SUP		THINNING	0	
		42	61	-	3 RD SUP		THINNING	0	
			60	-	1 ST SUP		THINNING	0	
			62	-	1 ST SUP		THINNING	0	
		43	49	-	3 RD AVB		THINNING	0	
			50	-	4 TH AVB		THINNING	0	
			54	-	3 RD AVB		THINNING	0	
			57	-	4 TH AVB		THINNING	0	
		44	61	-	1 ST SUP		THINNING	0	
			37	-	3 RD SUP		THINNING	0	
			49	-	3 RD SUP		THINNING	0	
			40	-	1 ST SUP		THINNING	0	
		45	40	-	1 ST SUP		THINNING	0	
			40	-	1 ST SUP		THINNING	0	
			40	-	1 ST SUP		THINNING	0	
		46	44	-	1 ST SUP		THINNING	0	
			44	-	3 RD SUP		THINNING	0	
			40	-	1 ST SUP		THINNING	0	
			40	-	1 ST SUP		THINNING	0	
			40	-	1 ST SUP		THINNING	0	
			40	-	2 ND AVB		THINNING	0	

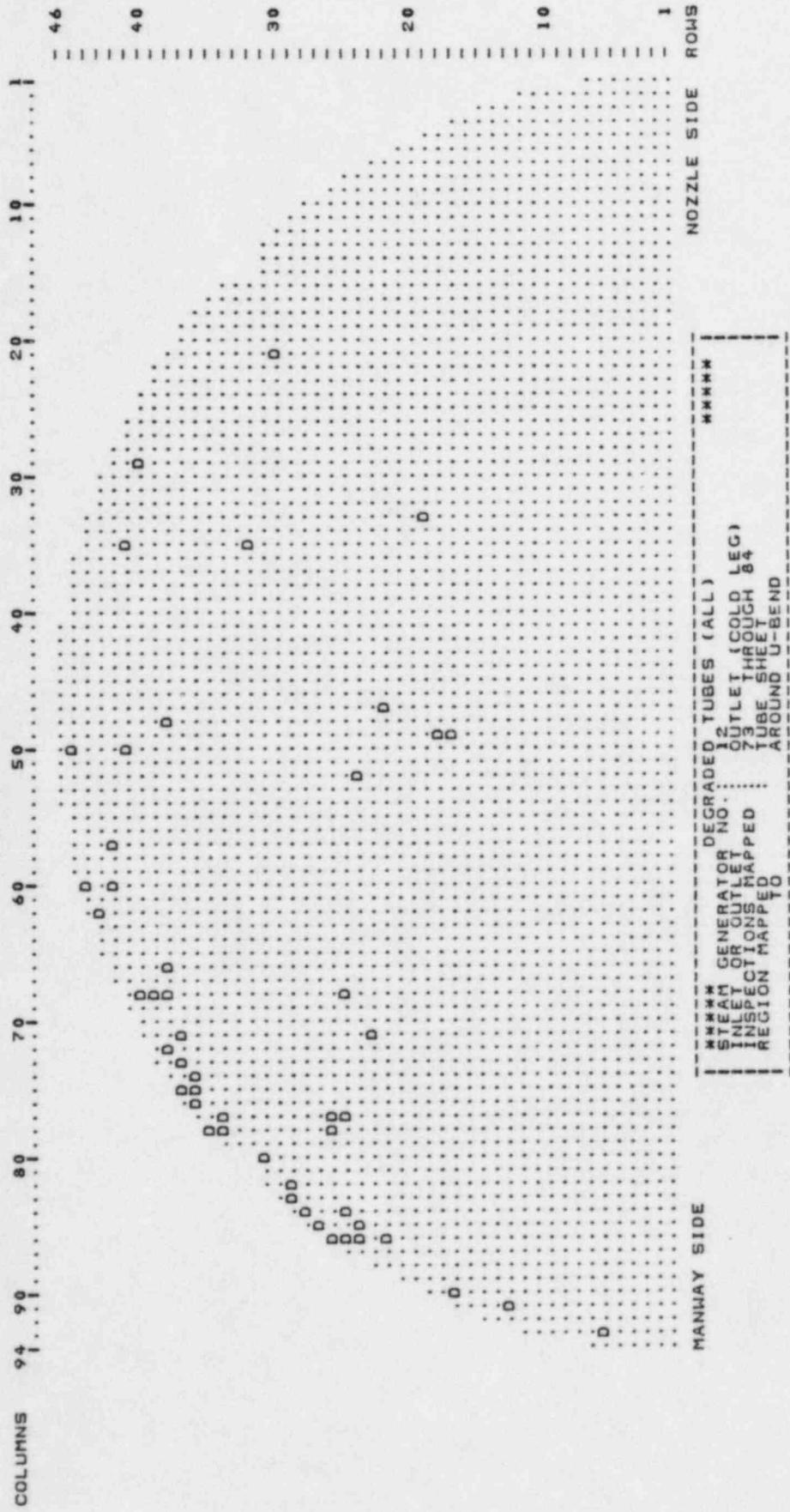
APPENDIX C

**STEAM GENERATOR NO. 12
EDDY CURRENT EXAMINATION RESULTS AND TUBE
SHEET MAPS**

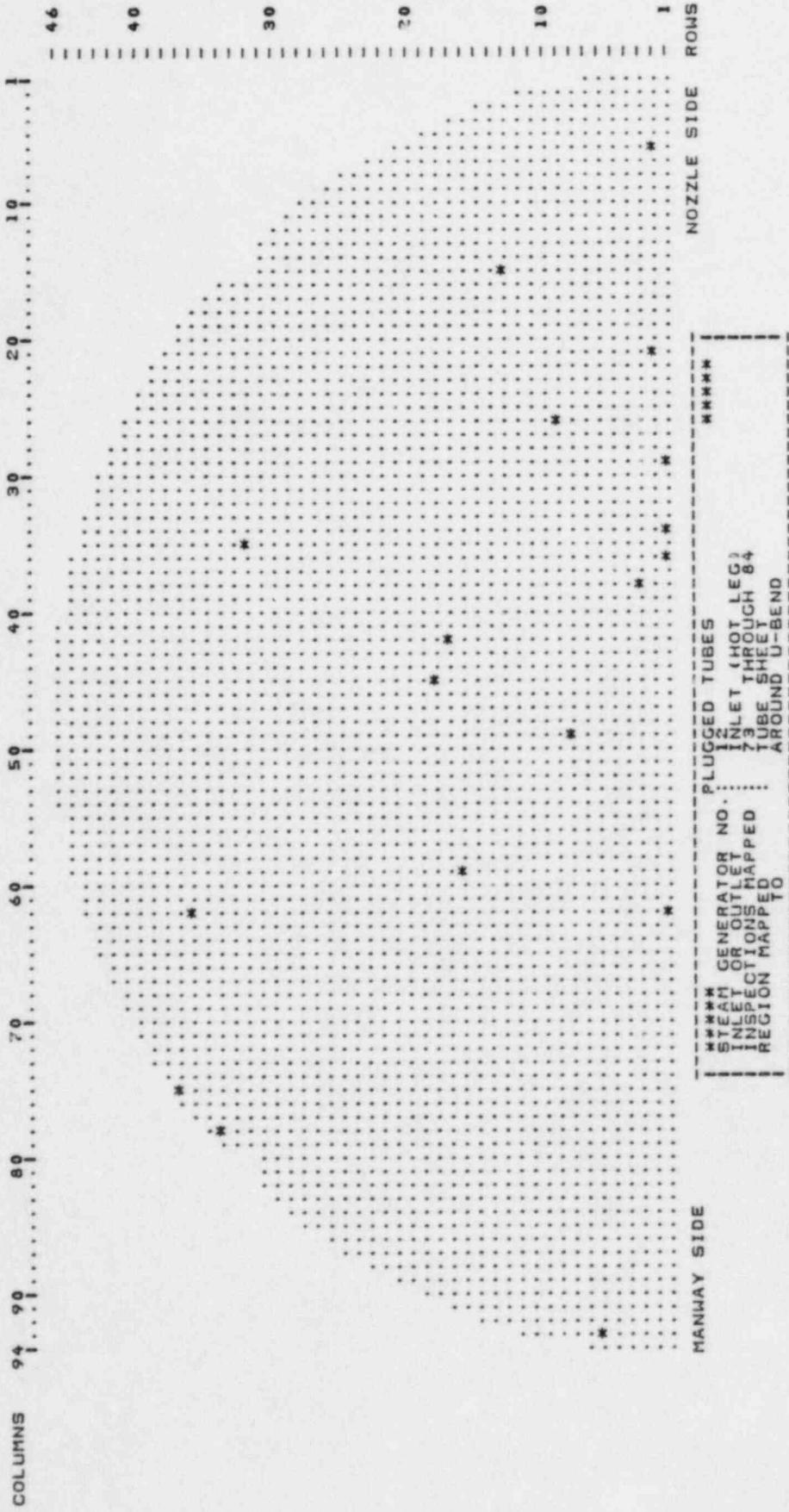
NORTHERN STATES POWER COMPANY
POWER PRODUCTION MATERIALS AND SPECIAL PROCESSES SECTION
PRAIRIE ISLAND NUCLEAR GENERATING PLANT STEAM GENERATOR TUBE MAP - WESTINGHOUSE SERIES 51



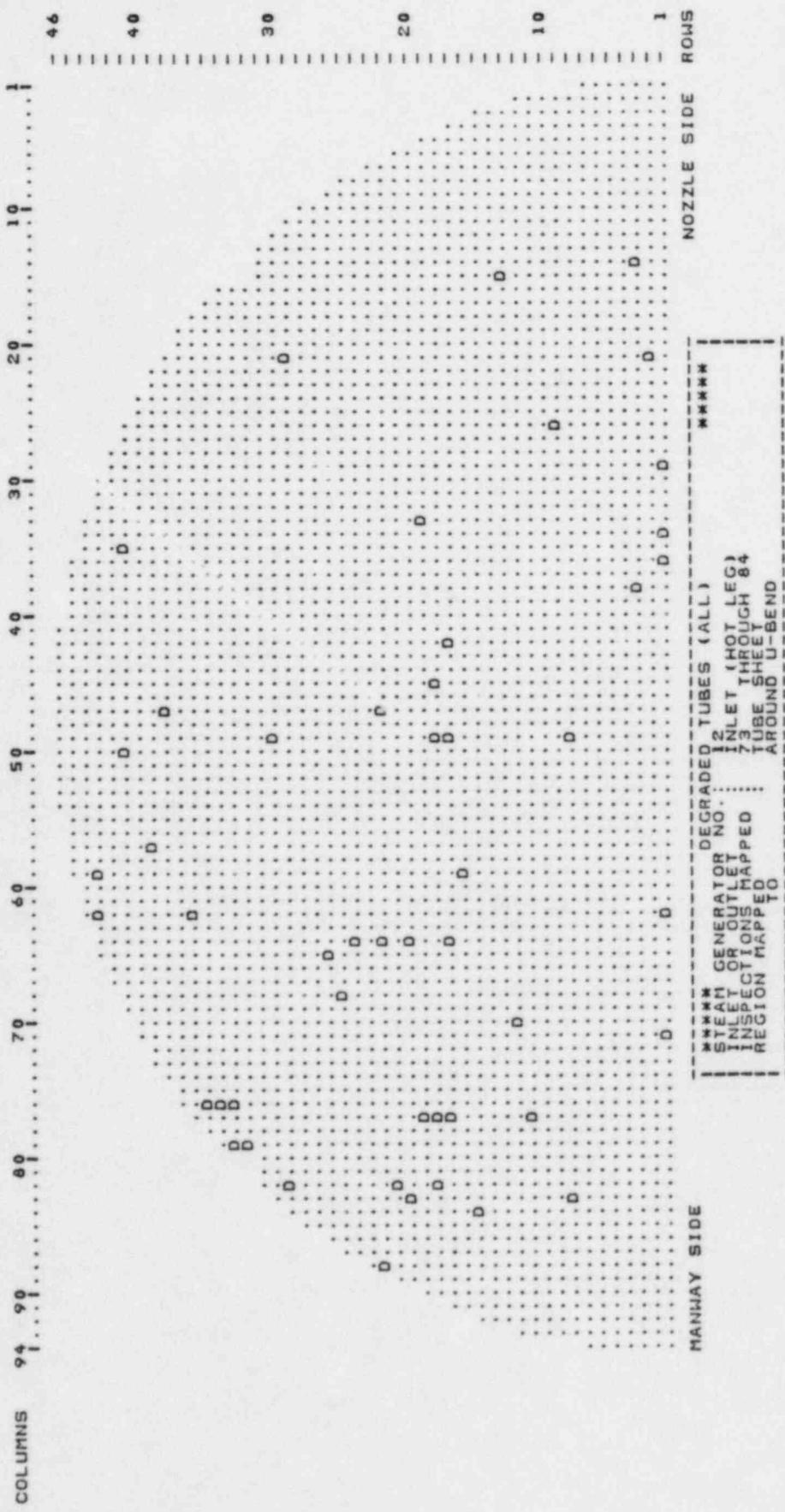
PRAIRIE ISLAND POWER PRODUCTION MATERIALS AND SPECIAL PROCESSES SECTION
NORTHERN STATES POWER COMPANY
GENERATOR TUBE MAP - WESTINGHOUSE SERIES 51



PRAIRIE ISLAND NUCLEAR GENERATING PLANT STEAM TUBE MAP - WESTINGHOUSE SERIES 51
NORTHERN STATES POWER COMPANY



PRAIRIE ISLAND NUCLEAR GENERATING PLANT STEAM GENERATOR TUBE MAP - WESTINGHOUSE SERIES 51
 NORTHERN STATES POWER COMPANY
 SPECIAL PROCESSES SECTION



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PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF PLUGGED STEAM GENERATOR TUBES

GEN NO.	SIDE	ROW	COL	YEAR	REMARKS/COMMENTS
12	INLET	1	2	83-	
			9	94-	
		2	6	94-	S/O
			6	94-	S/O
			4	94-	
	OUTLET	2	4	94-	
			4	94-	S/O
			7	94-	S/O
			6	94-	/O
			6	94-	/I

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PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
12	INLET	1	29	83-	12" BELOW	TUBE SH			
		34		83-	12" BELOW	TUBE SH	THINNING	98	
		36		83-	12" BELOW	TUBE SH	THINNING	<20	
		62		83-	12" BELOW	TUBE SH	THINNING	<20	
		71		83-	12" BELOW	TUBE SH	THINNING	<20	
		21		83-	11" BELOW	1ST SUP	THINNING	100	SQUIRREL
		14		83-	16" ABOVE	6TH SUP	THINNING	100	SQUIRREL
		38		83-	17" ABOVE	6TH SUP	THINNING	100	POSSIBLE BEND
8		49		83-	11" BELOW	TUBE SH	THINNING	100	
9		83		83-	10" BELOW	TUBE SH	THINNING	100	
12		26		83-	16" BELOW	4TH SUP	THINNING	100	
		77		83-		1ST SUP	THINNING	100	
		70		84-		1ST SUP	THINNING	100	
		15		84-	12" BELOW	2ND SUP	THINNING	100	
		64		84-		2ND SUP	THINNING	100	
16		59		84-	8" BELOW	TUBE SH	THINNING	100	
17		42		84-	11" BELOW	TUBE SH	THINNING	100	
		49		84-		2ND AVB	THINNING	100	
		64		84-		3RD AVB	THINNING	100	
		77		84-		3RD AVB	THINNING	100	
18		45		84-	10" BELOW	TUBE SH	THINNING	100	
		49		84-	11" BELOW	TUBE SH	THINNING	100	
		77		84-		2ND AVB	THINNING	100	
19		82		84-		2ND AVB	THINNING	100	
20		77		84-		3RD AVB	THINNING	100	
		64		84-		1ST SUP	THINNING	100	
		83		85-		1ST SUP	THINNING	100	
		47		85-		1ST SUP	THINNING	100	
		64		85-		1ST SUP	THINNING	100	
		24		85-		2ND SUP	THINNING	100	
		68		85-		1ST AVB	THINNING	100	
		25		85-		1ST AVB	THINNING	100	
		68		85-		1ST AVB	THINNING	100	
		26		85-		1ST AVB	THINNING	100	
		21		85-		1ST SUP	THINNING	100	
		82		85-		1ST SUP	THINNING	100	
30		49		85-	2" BELOW	1ST AVB	THINNING	100	
33		79		85-		2ND SUP	THINNING	100	
33		76		85-		3RD SUP	THINNING	100	
34		76		85-		3RD SUP	THINNING	100	
35		76		85-		3RD SUP	THINNING	100	
36		62		85-		2ND SUP	THINNING	100	
38		47		85-		TUBE SH	THINNING	100	
39		57		85-		2ND AVB	THINNING	100	
41		50		85-		4TH AVB	THINNING	100	
43		59		85-		4TH AVB	THINNING	100	
		62		85-		3RD AVB	THINNING	100	
		62		85-		1ST SUP	THINNING	100	
		93		85-		1ST SUP	THINNING	100	
OUTLET		6		84-	19" BELOW	TUBE SH	THINNING	100	

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PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
12	OUTLET	13	91	83-	1ST SUP		THINNING	22	
				84-	1ST SUP		THINNING	254	
17	49			84-	2ND AVB		THINNING		
				84-	3RD AVB		THINNING		
				90	1ST SUP		THINNING		
				84-	1ST SUP		THINNING		
18	49			84-	1ST AVB		THINNING		
				84-	2ND AVB		THINNING		
19	33			84-	2ND AVB		THINNING		
				84-	3RD AVB		THINNING		
				84-	4TH AVB		THINNING		
22	47			84-	3RD AVB		THINNING		
				84-	1ST SUP		THINNING		
23	77			84-	2ND AVB		THINNING		
N4	57			84-	2ND SUP		THINNING		
				84-	3ST SUP		THINNING		
				84-	3ST SUP		THINNING		
				84-	3ST SUP		THINNING		
25	68			84-	3ST SUP		THINNING		
				77	TUBE AVB		THINNING		
				84-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
26	77			84-	2ST SUP		THINNING		
				78	TUBE SH		THINNING		
				84-	2ST SUP		THINNING		
27	86			84-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
28	84			84-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
29	82			84-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
30	21			75	20 " ABOVE		THINNING		
31	80			82	2ST SUP		THINNING		
				83	2ST SUP		THINNING		
				84-	1ST SUP		THINNING		
32	35			84-	2ND AVB		THINNING		
				84-	3RD AVB		THINNING		
34	77			83-	1ST SUP		THINNING		
				84-	1ST SUP		THINNING		
				78	1ST SUP		THINNING		
				82-	1ST SUP		THINNING		
35	78			84-	1ST SUP		THINNING		
36	74			83-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
				75	2ST SUP		THINNING		
				83-	2ST SUP		THINNING		
				76	2ST SUP		THINNING		
				83-	2ST SUP		THINNING		
37	71			83-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
				73	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		
				75	2ST SUP		THINNING		
				83-	2ST SUP		THINNING		
38	48			84-	2ST SUP		THINNING		
				84-	2ND AVB		THINNING		
				66	2ND AVB		THINNING		
				68	2ST SUP		THINNING		
				72	2ST SUP		THINNING		
				83-	2ST SUP		THINNING		
39	68			84-	2ST SUP		THINNING		
				82-	2ST SUP		THINNING		
				83-	2ST SUP		THINNING		
				84-	2ST SUP		THINNING		

DEC 31, 1984

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
LIST OF IMPERFECT, DEGRADED, AND DEFECTIVE TUBES

GEN NO.	SIDE	ROW	COL	YEAR	FROM	TO	DEFECT OR OBS	%	REMARKS/COMMENTS
12	OUTLET	40	29	82-	6TH SUP	7TH SUP	THINNING	0	BETWEEN
				83-	6TH SUP		THINNING	<	
				81-	6TH SUP		THINNING	<	
				68	1ST SUP		THINNING	>	
				84-	1ST SUP		THINNING	>	
41	35	84-			4TH AVB		THINNING	>>	
42	57	83-		34 " ABOVE	3RD AVB		THINNING	>>	
				83-	3RD SUP		THINNING	>>	
				82-	4TH SUP		THINNING	>>	
				60	1ST SUP		THINNING	>>	
43	62	83-			1ST SUP		THINNING	>>	
44	60	84-			1ST SUP		THINNING	>>	
45	50	84-			1ST SUP		THINNING	>>	
				18" BELOW					

APPENDIX D

**FORM NIS-1, OWNERS DATA REPORT FOR
INSERVICE INSPECTION**

FORM NIS-1 OWNERS' DATA REPORT FOR INSERVICE INSPECTIONS

(As Required by the Provisions of the ASME Code Rules)

1.) Owner NORTHERN STATES POWER COMPANYAddress 414 NICOLLET MALL, MINNEAPOLIS, MN 554012.) Plant PRAIRIE ISLAND NUCLEAR GENERATING PLANTAddress WELSH, MN3.) Plant Unit 1 4.) Owner (Certificate of Authorization) - - -5.) Commercial Service Date 12-16-73 6.) National Board Number for Unit - - -

7.) Components Inspected

<u>Component or Appurtenance</u>	<u>Manufacturer or Installer</u>	<u>Manufacturer or Installer Serial No.</u>	<u>State or Province No.</u>	<u>National Board No.</u>
STEAM GENERATOR TUBING				
STEAM GENERATOR				
#11	WESTINGHOUSE	1101	MN	6824
STEAM GENERATOR				
#12	WESTINGHOUSE	1102	MN	6825

FORM NIS-1 (back)

8.) Examination Dates Oct. 23 to Nov. 4, 1984.) Inspection Interval Dec., 1973 to Dec., 1983

10.) Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval.

EXAMINED VARIOUS TUBES IN EACH GENERATOR. EXAMINATIONS FOCUSED ON TURING ADJACENT TO LEAKERS AND IN HISTORICALLY PREVALENT DEGRADED AREAS.

11.) Abstract of Conditions Noted.

MINIMAL OVER-ALL DEGRADATION DETECTED WITH EXCEPTIONS BEING TUBES WHICH EXHIBITED LEAKS VISUALLY.

12.) Abstract of Corrective Measures Recommended and Taken.

ALL DEFECTIVE TUBING WAS REMOVED FROM SERVICE VIA MECHANICAL PLUGGING OF THE TUBE ENDS.

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 1/9/1984 Signed NORTHERN STATES POWER by Michael J. Anderson
Owner

Certificate of Authorization No. (if applicable) N/A Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of MINNESOTA and employed by HARTFORD STA. BOILER LTD. of HARTFORD, CONN. have inspected the components described in this Owner's Data Report during the period 10-23-84 to 11-4-84, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owners' Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date Jan. 9 19 85

R.J. Kuykendall
Inspector's Signature

Commissions 1V79904 MN 85-34
National Board, State, Province & No.

ATTACHMENT I

NRC COMMITTMENt REFERENCE

MLA121384WMH01-LT



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20585

Attachment I
Page 1 of 1

JUL 6 1984

Docket No. 50-282

Mr. D. M. Musolf
Nuclear Support Services Department
Northern States Power Company
414 Nicollet Mall
Midland Square - 4th Floor
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

We have completed our review of your reevaluation of the eddy current test results transmitted by letter dated June 18, 1984. The eddy current test results are from the December 1983 steam generator tube inspection data of the Prairie Island Generating Plant Unit No. 1. This reevaluation was prompted by a small primary to secondary leak (approximately 0.015 gpm) detected in February 1984.

Based on our review of your submittal we conclude that, since the identified defects are located within the tubesheet of the steam generator, they do not at this time constitute an undue hazard to public health and safety. On this basis, the Prairie Island Nuclear Generating Plant Unit No. 1 can continue to operate provided that the primary to secondary leakage does not exceed 0.3 gpm. Based on your commitment, if the 0.3 gpm leak rate is exceeded, the unit must be shut down and the defective tubes repaired. In addition, you are requested to inform us of your plans for corrective action if leak rate exceeds 0.15 gpm.

Since your reevaluation has uncovered three defective tubes, an inconsistency exists with the technical specification, TS 4.12.D.2. In order to correct this matter, you are requested to submit a technical specification change request by July 9, 1984 so that a one-time amendment to TS 4.12.D.2 would permit continued operation with the three defected tubes in service until the end of the current cycle (December 1984).

Our Safety Evaluation is enclosed.

Sincerely,

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosure:
As stated

cc: See next page