



UNITED STATES  
ATOMIC ENERGY COMMISSION  
DIVISION OF COMPLIANCE  
REGION I  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

201 645. 3960

NOV 9 1971

R. T. Carlson, Sr. Reactor Inspector  
Division of Compliance, Region I

CO INSPECTION REPORT NO. 50-219/71-03  
JERSEY CENTRAL POWER & LIGHT COMPANY  
OYSTER CREEK 1 - BWR

The inspection conducted on October 7 and 8, 1971, which was limited to reviewing control of personnel exposures, and radiation and contamination control during the current extended outage, showed no significant deficiencies in their program. Exposure control was maintained on a daily basis with a central control mechanism. Radiation levels in the major work areas were generally low. High radiation areas were well delineated and spoken to in the RWP's covering those areas. All work was controlled by RWP's covering those areas. All work was controlled by RWP's in which work conditions and requirements were well defined. Contamination control appeared adequate.

The radiation protection staff had been increased by six additional technicians provided by contract from General Electric and Eberline. The coverage appeared to be adequate to cope with outage problems, however it may have been spread a little thin during the early stages of the outage.

Two air balance problems noted by the inspector during the inspection apparently had not generated any problems; however, they did provide a potential for contamination spreads or high air concentrations outside of radiation zones. Corrective actions to prevent recurrence were initiated.

The licensee currently has no capability to monitor air concentrations on a continuous basis. Licensee has recognized this and is currently investigating their needs. They do sample continuously with the results not being indicative of any problems to date; however the potential for exposure to excessive air concentrations does exist. Licensee agreed to investigate the needs for a bioassay program. It appears that the current outage has provided experience that will be applied to future outages.

*R. T. Carlson - JR*  
R. J. Meyer  
Radiation Specialist



U. S. ATOMIC ENERGY COMMISSION  
DIVISION OF COMPLIANCE  
REGION I

Field Notes for:

CO Inspection Report No. 50-219/71-03

Subject: Jersey Central Power & Light Company

Oyster Creek 1

License No. DPR-16

Location: Forked River, N. J.

Priority

Category C

Type of Licensee: BWR

Type of Inspection: Special, Announced

Dates of Inspection: October 7, 1971

Dates of Previous Inspection: June 22-25 & July 2, 1971

Principal Inspector: T. Young, Jr.

Date

Accompanying Inspectors:

Date

Date

Other Accompanying Personnel:

Date

Reviewed By: R. T. Carlson

11/8/71  
Date

Proprietary Information:



OC1

A. Persons Contacted

1. ~~T.~~ T. McCluskey Station Superintendent.
2. D. Ross Technical Supervisor
3. D. Reeves Technical Engineer
4. E. Riggle Maintenance Supervisor
5. J. Sullivan Asst. Technical Engineer
6. J. P. Zilinskas GE Manager
7. W. L. Swanson GE Shift Supervisor

B. Operations

1. Shipping complete (12 <sup>bundles</sup> not shipped)  
44 Leakers detected  
24 New fuel bundles at the site -  
and will be placed around  
the periphery in the core.  
20 of the leakers to be  
reconstituted and placed in  
the high flux area of the  
core.  
11 fuel bundles have been inspected  
to date.
2. 8 LPKM Chambers are being  
replaced.



3. Procedures reviewed

- Ⓐ General Information  
Fuel Rod Inspection & Bundle  
Reconstitution, JCP&L Co.  
OC Power Station

Prepared by:

Nuclear Fuel Dept.

GE Co. San Jose Calif.

J. P. Zilinskas, Jul 23, 1971

- Ⓐ Proprietary Information

- Ⓑ Fuel Reconstitution Procedure  
JCP&L Co, OC Power Station

Prepared by:

Nuclear Fuel Dept.

GE Co, San Jose Calif

JP Zilinskas, Jul. 23, 1971

- Ⓐ Proprietary Information

- Ⓐ Both Procedures were reviewed  
and approved by JCP&L Co. on  
9/29/71.



© ~~©~~ Reactor fuel examination & reconstruction

Prepared by:

J. S. Atkinson Specialist - Field Service.  
W. L. Swanson " " "  
J. P. Zilinski " " "

Approved by:

KW Hess, Manager - Field Services

Aug 71  
⊙ Proprietary Information

C. Observation of operations

1. All ruffling was done by ~~GE~~ A Personnel.
2. Inspection is done by GE Personnel with a GE and a ~~GE~~ Checker to check the rod's position out and placement into the bundles.
  - a) When the rod is taken out of the bundle it is first run through a brush cleaner and then run through the inspection station.
  - b) While in the inspection station the rod is checked over 100% of its length with ultrasonics & eddy currents.



The Ultrasonic Test is for water inside the fuel rod and the eddy current test is for flaws in the cladding.

3. All fuel bundle handling is done by ~~OC~~ personnel.

a) Fuel movement from core to spent fuel pit (SFP)

b) Fuel bundle movement from SFP to fuel preparation racks.

c) Removal, storage and replacement of channels and storage of channel hardware.

d) Fuel movement from fuel preparation racks to SFP. and,

e) Fuel movements from SFP to the reactor core.

4. All of the fuel rod cleaning, inspection and reconstitution is done by GE personnel in the fuel preparation racks.

5. Independent records are ~~kept~~ kept by the ~~OC~~ and the GE. Checkpoints of the fuel rod removal and the fuel rod placement back into the bundle.



6. When the dechanneled fuel assembly is placed into the fuel preparation machine, the GE supervisor and the ~~OC~~ checker verified the assembly identification and orientation in the rack so that fuel rods were readily identifiable upon removal and replacement. This information was recorded on the check sheets.

7. The ~~OC~~ Personnel were in constant communication (via intercom) with the Control Room from the fuel handling crane platform. (3 Men) The fuel handler supervisor, who was also the tool operator was a licensed senior operator. In the inspector's opinion very alert and aware of what was going ~~on~~ on.

(\*) There was a check sheet taped to the bridge telling the operator of which fuel bundle to move next and where to place it. The checker on the bridge recorded this information on his check sheet. All direction for fuel <sup>bundle</sup> movement



Came from the control room and  
logged in the console log. The fuel  
handler's check sheet was only to  
insure that the information  
relayed from the control room  
was correct.

An up to date record board  
was kept in both the control  
room and in the Reactor Building  
fuel room area, depicting all fuel  
bundles by identification numbers  
and their placement, whether  
in the core, in the SFF or in the  
preparation racks.

8. The weak point in the over all  
operation were the lack of nuclear  
experience of the four GE rod  
handlers. (The GE SS and the manager,  
have well grounded backgrounds)  
They are technicians (non nuclear  
trained) from a GE service shop  
They were given one week of  
training <sup>on the way</sup> the tools using a  
dummy fuel bundle and ~~some~~  
some training on the hazards



of radiations. This was a controlled operation with the GE shift supervisor looking over the fuel handlers shoulders most of the time. However on one occasion the inspector witnessed that one of the GE fuel handlers, hung four (4) ~~fuel~~ fuel handling tools on one hook. This was too much weight for the hook and it partially straighten out allowing two (2) of the tools to fall into the SFP. The tools landed in a spent fuel rack. Fortunately no fuel was in the rack. This was pointed out ~~to the SFP~~ to Mr McCluskey at the exit interview.

D. Exit Interview

An exit interview was held with Mr McCluskey and concern was expressed with the lack of nuclear experience of the GE fuel handlers and their backgrounds. The incident of the fuel handling tools was



pointed out as an example which  
may have resulted in a very  
serious problem if fuel had been  
stored <sup>in</sup> those particular racks. Mr  
McCluskey stated that this was  
also a concern of Cyston Creek  
management and that their ~~the~~  
(OC) checker was an experienced  
licensed operator with orders  
to report any ~~unsafe~~ unsafe  
operations to management.



U. S. ATOMIC ENERGY COMMISSION  
DIVISION OF COMPLIANCE  
REGION I

Field Notes for:

CO Inspection Report No. 50-219/71-03

Subject: Jersey Central Power & Light Company

Oyster Creek 1

License No. DPR-16

Location: Forked River, N. J.

Priority

Category C

Type of Licensee: BWR

Type of Inspection: Special, limited, announced

Dates of Inspection: October 7 and 8, 1971

Dates of Previous Inspection: June 22-25 and July 2, 1971

Principal Inspector: R. J. Meyer

Date

Accompanying Inspectors:

Date

Date

Other Accompanying Personnel:

Date

Reviewed By: R. T. Carlson

11/8/71  
Date

Proprietary Information:



## A. Persons Contacted

T. McCluskey, Station Superintendent

D. Ross, Technical Supervisor

D. Kaulback, Radiation Protection Supervisor

R. Standnour, Radiation Protection Supervisor

A. Grimes, General Foreman (Senior)

## B. Administration & Organization

Ross stated that no significant changes have occurred in the area of responsibility and that he has been involved with that responsibility since the outage. Mr. McCluskey, Sta. Supt. D. Kaulback, Radiation Protection Supervisor, reporting to D. Ross.

As directed by Ross, additional personnel were added to the Rad. Protection staff for the outage. Robert Standnour (waste management systems) was moved in to provide additional support. Standnour, reporting to Kaulback. Six additional Rad. Techn. have been added to the staff for the outage, by contract with Eberline and Electric. Responsibility for the outage work has been split between Kaulback and



standdown, for purposes of greater exposure coverage.

### C. Operations

Reactor down for extended outage 9/27

### D. Through 0

Not inspected - Inspection limited to  
visual check of the reactor building and  
surrounding area.

Reactor down for extended outage 9/27

### E. External

A review of the exposure records  
exposure histories were documented on Form 9  
and 6 equivalent. JCP&H provides a  
and packet dosimeters and maintains a  
record of exposure as determined by the  
packet dosimeters. Film badges remain on a  
monthly exchange frequency.

Pass started that all external dosimeters







are taken on a continuous basis and on a grab basis in other samples for potential. Continuous samples are taken in the deep well area, and in the bottom at the same time as the grab samples. Air is being taken in the same way, and more gases. Gas sampling is done on a grab basis at least twice a shift, and occurs when air bubbles are noted in the water.

It is noted that during the period of the study, the air and at the same time, the air is being taken at the same time, independent of the air sampling. The air sampling was used as a guide for the air sampling. Respirator / protection and described by procedure 003.12. The air by record air concentration and the air those concentrations allowed for the air sampling. The air sampling was used as a guide for the air sampling.

In general concentrations were low, and the rate of air concentration was not at the same time, but it was at the same time. The air sampling was used as a guide for the air sampling.



creating over the course of the day. I am  
not sure that remains how we do not  
presenting much, just a few more.

Currently licensee has no capabilities in continuously monitoring air quality. This may depend on continuous sampling with auto fault determination. Rose stated that neither budget has money for continuous air monitoring equipment on the new investment air needs.

3. Reddish-brown on the underside of the leaves



As evidenced by records, statements and observations by the inspector, radiation and contamination levels appear to be well controlled. The entire area was well defined, posted and controlled by barriers, doors, attending personnel, and access log sheets. Routine surveys were being conducted and results recorded. High radiation areas were clearly marked and access was restricted to appropriate personnel and equipment. The work area, as a whole, was well controlled.

The results of the survey indicate that the work area, in general, is in good control. It was noted that significant contamination levels were not observed. It would contribute to exposure problems if airborne problems had been observed under controlled conditions and supervision of H.P. personnel. In general, the work area was well controlled. The work locations were noted to be as determined by survey equipment. According to statements and records kept, control measures had been implemented, and control measures immediately apparent to the control



areas. Losses of control were attributed to  
careless removal and control of protective equip-  
ing, and generally poor step off pad site in the  
Eberline Red. Technicians stated that the  
thought was emphasis given to people  
during the training given to continue to people  
but that was gradually getting them accustomed  
to good practices. Ross stated that the train-  
ing would be given in the future.

#### 1. Air Release

Upon exit from the Tower at the air release  
tower a two by observation paper was placed  
on noted that the air flow appeared to be con-  
st. the drywell area. This was verified by measuring  
a 10 ft. tall. Ross stated that the air  
should be into the drywell. A report was made  
to Ross. The ventilation system was checked  
and the air flow problem was corrected. The  
cause. Ross stated that during the vessel re-  
removal some air flow adjustment had been  
made to occur. That an adequate <sup>air</sup> flow could  
be maintained down and across the tank and  
over the working personnel. He stated that it  
apparently had not been changed since it was  
the same time in the air release tower.







be used for a specific job that was abnormal  
or of an unusual nature. RWP's were  
by the Red Tech (ICR or only) or the Red  
Photo. Supervisor and signed. All RWP's must be  
counter signed by the Over Shift Foreman. The  
RWP's define the requirements for, monitoring  
clothing, dosimeters, and respiratory protective  
Description of work allowed is defined in these  
instructions as to hazards, areas to avoid, and  
SOS rates are given. Instructions should be  
written to work under a specific RWP.

It is noted that the RWP's by themselves do not  
that all RWP's are well defined, well  
detailed, appropriate signatures in full,  
defined with proper maintenance, and  
location and in the Red. First of all,  
noted that upon job completion RWP's are  
and removed from service, and filed.

#### 6. Notes

It is noted that the RWP's were in full and  
appropriate to the needs.

It is noted that the RWP's were in full and



Friskars, portable G.M.'s, and hand operated Radio  
for personal surveys were in good supply.

It was noted that a Rad. Tech was in attendance  
or available to the major work areas.



OCT 26 1971

J. G. Keppler, Chief, Reactor Testing & Operations Br.  
Division of Compliance, HQ

CO INQUIRY REPORT NO. 50-219/71-07  
JERSEY CENTRAL POWER & LIGHT COMPANY  
OYSTER CREEK - BWR  
CONSTRUCTION UNIONS PICKETING TO PROTEST THE  
USE OF GE EMPLOYEES FOR MAINTENANCE WORK

The subject inquiry report is forwarded for information.

The plant was shut down for removal of poison curtains, fuel replacement/reconstitution, and a turbine warranty inspection at the time. No safety problems were identified with respect to this event.

No further action is planned by Region I.

R. T. Carlson  
Senior Reactor Inspector

Enclosure:  
Subject Inquiry Report

cc: E. G. Case, DRS (3)  
R. S. Boyd, DRL (2)  
R. C. DaYeung, DRL (2)  
D. J. Skovholt, DRL (3)  
H. R. Denton, DRS (2)  
A. Giambusso, CO  
L. Kornblith, CO  
R. H. Engelken, CO  
Regional Directors, CO  
DR Central Files

~~8344-210170 (14)~~ B/406

OFFICE ▶	CO <i>232</i>	<i>RC</i>			
SURNAME ▶	Cantrell:smg	Carlson			
DATE ▶	10/26/71				



CO Inquiry Report No. 50-219/71-07

Subject: Jersey Central Power & Light Company

License No.: DPR-16

Facility: Oyster Creek (BWR)

Title: Construction Unions Picketing to Protest the  
Use of GE Employees for Maintenance Work

Prepared by: F. S. Cantrell, Reactor Inspector

Date

A. Date and Manner AEC was Informed:

In telephone conversation with Plant Superintendent T. McCluskey at 9:00 am, October 26, 1971.

B. Description of Particular Event or Circumstance:

At 6:00 am October 23, 1971, the Monmouth County - Ocean County Building Trades Association put out pickets at the entrances to the Oyster Creek Nuclear Generating Station protesting the use of General Electric Company employees to perform maintenance and repair work. The normal day shift crew reported for work at 4:00 am as scheduled, but a special crew for handling fuel was turned back at 8:00 am as well as GE employees performing warranty work on the main turbine and the core spray system check valves. Discussions were held between JCP&L and the Building Trades representatives. Ten maintenance, supervisory and technical personnel were allowed in the plant at 2:00 pm. The 4 - 12 shift was permitted to enter and the pickets were withdrawn at 6:00 pm. The net effect was approximately 1 day lost time for the outage. Mr. McCluskey stated that the plant always had at least the required number of licensed personnel to run the plant and no safety problems were encountered.

C. Action by Licensee:

No further action is planned at this time. According to Mr. McCluskey, JCP&L Co. does not normally use construction trades people in any of their plants for maintenance and repair work.

~~8344010176~~ (IP)



OCT 7 1971

J. G. Keppler, Chief, Reactor Testing & Operations Br.  
Division of Compliance, HQ

CO INQUIRY REPORT NO. 50-219/71-06  
JERSEY CENTRAL POWER & LIGHT CO. (OYSTER CREEK - BWR)  
CO INSPECTOR REVIEW OF RESULTS OF FUEL SIPPING -  
LICENSEE REFUELING PLANS

The attached inquiry report is forwarded for information in advance  
of a detailed inspection report.

Preliminary results of our inspection indicate that the licensee's  
refueling activities are proceeding according to plans with no  
major difficulties having been encountered to date.

We intend to continue following closely the current outage and  
will keep you informed as is appropriate.

R. T. Carlson  
Senior Reactor Inspector

Enclosure:  
Inquiry Rpt 50-219/71-06

cc: E. G. Case, DRS (3)  
R. S. Boyd, DRL (2)  
R. C. DeYoung, DRL (2)  
D. J. Skovholt, DRL (3)  
H. R. Denton, DRS (2)  
A. Giambusso, CO  
L. Kornblith, CO  
R. Engelken, CO  
Regional Directors, CO  
DR Central Files

~~8304210180~~ 8/407

OFFICE ▶	<i>CO</i>	<i>DR</i>				
SURNAME ▶	Cantrell:smg	Carlson				
DATE ▶	10/7/71					



CO Inquiry Report No. 50-219/71-06

Subject: Jersey Central Power & Light Company

License No.: DPR-16

Facility: Oyster Creek 1 (BWR)

Title: CO Inspector Review of Results of Fuel Sipping -  
Licensee Refueling Plans

Prepared by: F. S. Cantrell, Reactor Inspector

Date

A. Date and Manner AEC was Informed:

October 7, 1971 during a telephone conversation with Tolbert Young, Jr., Reactor Inspector, who is currently at the site reviewing refueling activities.

B. Description of Particular Event or Circumstance:

Mr. Young reported that the licensee has completed the planned fuel sipping of the reactor core. All but 12 assemblies on the periphery were sipped. The results indicate 44 of the 548 assemblies sipped contain one or more leaking fuel pins. All of the identified leakers were located in the high flux region (central) of the core. The licensee believes that the probability of the remaining 12 assemblies being leakers is very low.

C. Action by Licensee:

According to Mr. Young, the licensee plans to replace 24 of the leaking assemblies with assemblies from the periphery of the core, with the latter assemblies being replaced with new fuel assemblies. The remaining 20 leaking assemblies will be reconstituted, replacing identified leaking fuel pins with non-leaking pins from the removed assemblies as necessary.

8244214189 (1p)



SEP 10 1971

J. G. Keppler, Chief, Reactor Testing and Operations  
Branch, Division of Compliance, HQ

## EVALUATION MEMORANDUM

CO INQUIRY REPORT NO. 50-219/71-05

JERSEY CENTRAL POWER &amp; LIGHT CO. (OYSTER CREEK - BWR)

ISOLATION CONDENSER RELAY FAILURE

The attached Inquiry Report relating to the failure of time delay relays associated with the isolation condenser is forwarded for information. The action taken by the licensee appears to be adequate.

It is recommended that a survey of other BWR licensees be conducted to determine their experience with this type relay. From the amount of trouble JC has experienced, it would appear that this relay is not reliable for this type service. We plan to review this subject with Nine Mile Point and Millstone. We will keep you informed as appropriate.

R. T. Carlson  
Senior Reactor Inspector

## Enclosure:

CO Inquiry Rpt. No. 50-219/71-05

cc: E. G. Case, DRS (3)  
R. S. Boyd, DRL (2)  
R. C. DeYoung, DRL (2)  
D. J. Skovholt, DRL (3)  
H. R. Denton, DRS (2)  
A. Giambusso, CO  
L. Kornblith, CO  
R. H. Engelken, CO  
Regional Directors, CO  
DR Central Files

~~8344210494~~ (10)

B/408

OFFICE ▶	CO				
SURNAME ▶	Cantrell:smg	Carlson			
DATE ▶	9/10/71				



CO Inquiry Report No. 50-219/71-05

Subject: Jersey Central Power & Light Co.

Licensee No.: DPR-16

Facility: Oyster Creek (BWR)

Title: Isolation Condenser Relay Failure

Prepared by: F. S. Cantrell, Jr., Reactor Inspector

Date

A. Date and Manner AEC Was Informed:

1. Mr. Tom McCluskey, Station Superintendent, reported the matter by phone to Region I on September 8, 1971. Additional information was obtained in phone conversations with Messrs. McCluskey and Joe Carroll, Operations Supervisor, on September 9, 1971.

B. Description of Event:

2. On September 3, 1971 while conducting a surveillance test to insure isolation of the two redundant isolation condensers upon detection of a line break, the two relays on B isolation condenser, which initiate isolation in the event of a line break, would not operate. (Previous failures of isolation condenser relays were reported to DRL in letters from I. R. Finfrock dated May 17 and July 21, 1971.)
3. The relays that activate the isolation condenser were still operable. In the event of a line break, the condensers could have been isolated manually.

C. Action by Licensee:

4. Due to the physical location of the relays in the control board, the failed relays were not removed; however, the wiring was disconnected from the old relay (GE CR 120) and a replacement time delay relay (Agastat 7022PD) was installed at an accessible location. The Agastat relay performs the same function. All eight of the GE type relays associated with the isolation condensers have now been replaced with the Agastat relays.
5. One of the relays that failed previously was examined and the cause of failure was determined to be differential expansion resulting from overheating. The heat source is a resistor in the relay. The differential expansion physically prevented operation of the relay.

~~8344-10493~~ (2B)



--2--

6. Mr. Carroll stated that JC is reviewing their instrumentation to see if the GE relay is used in other services at the plant.
7. Mr. McCluskey stated that JC would submit a written report of the most recent failure to DRL in accordance with the requirements of the license.