Docket Nos.: 50-440
and 50-441

OCT 22 1984

Mr. Murray R. Edelman
Vice President - Nuclear Group
The Cleveland Electric Illuminating Company
P. O. Box 5000
Cleveland, Ohio 44101

Dear Mr. Edelman:

Subject: Request for Additional Information Pertaining to the Perry Nuclear Power Plant Conformance with TMI Action Plan Item II.D.1, Testing of Safety - Relief Valves -- SER Confirmatory Issue (7)

In Section 1.10 of SSER 4, we reported that Perry's conformance with TMI Action Plan Item II.D.1 was under review by the staff. The enclosed information is requested in order for the staff to complete its review. The need for this information has arisen from the staff's review of the generic safety-relief valve test results contained in General Electric Report NEDE-24988-P, which must be addressed on a plant-specific basis by CEI. The enclosure generally indicates the issues or concerns that should be addressed in your response to justify the applicability of those generic test results to Perry.

Your responses should be identified as addressing Q271.01 through Q271.04 for eventual documentation in the FSAR; however, you should confirm that this numbering sequence is corrected before responding, and the Perry Project Manager advised accordingly. Your staff should also inform the Project Manager when we may expect to receive your responses within 7 days after receipt of this letter.

Sincerely,

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Enclosure:
As stated

cc: See next page

CONCURRENCES:

DL:LB#1	DL:LB#1
JStefano:es	BJYoungblood
10/19/84	10/19/84

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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As stated

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PERRY

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Enclosure

Request for Additional Information by the
Equipment Qualification Branch

TMI Action Plan II.D.1

Prior submittals do not provide the basis for the conclusion that the test results presented in NEDE-24988-P on safety/relief valve testing are applicable to your specific plant. Describe the basis thoroughly, as indicated below.

271.01. The test program utilized a "rams head" discharge pipe configuration. Most plants utilize a "tee" quencher configuration at the end of the discharge line. Describe the discharge pipe configuration used at your plant and compare the anticipated loads on valve internals in the plant configuration to the measured loads in the test program. Discuss the impact of any differences in loads on valve operability.

271.02. The test configuration utilized no spring hangers as pipe supports. Plant specific configurations do use spring hangers in conjunction with snubber and rigid supports. Describe the safety relief valve pipe supports used at your plant and compare the anticipated loads on valve internals for the plant pipe supports to the measured loads in the test program. Describe the impact of any differences in loads on valve operability.

271.03. The purpose of the test program was to determine valve performance under conditions anticipated to be encountered in the plants. Describe the events and anticipated conditions at the plant for which the valves are required to operate and compare these plant conditions to the conditions in the test program. Describe the plant features assumed in the event evaluations used to scope the test program and compare them to the features at your plant. For example, describe high level trips to prevent water from entering the steam lines under high pressure operating conditions as assumed in the test event and compare them to trips used at your plant.

271.04. Describe how the values of valve C_v's in report NEDE-24988-P will be used at your plant. Show that the methodology used in the test program to determine the valve C_v will be consistent with the application at your plant.