

OYSTER CREEK NUCLEAR GENERATING STATION

PROVISIONAL OPERATING
LICENSE NO. DPR-16

TECHNICAL SPECIFICATION
CHANGE REQUEST NO. 123, Rev. 1
DOCKET NO. 50-219

Applicant submits by this Technical Specification Change Request No. 123, Rev. 1 to the Oyster Creek Nuclear Generating Station Technical Specifications, modified pages 4.3-1, 4.3-2, 4.3-3 and 4.3-4.

By *Peter B. Fiedler*
Peter B. Fiedler
Vice President and Director
Oyster Creek

Sworn and subscribed to before me this *11th* day of *February*, 1985.

Diana A. Maldet
Notary Public of New Jersey

DIANA A. MALDET
A Notary Public of New Jersey
My Commission Expires June 5, 1986

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UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

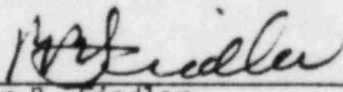
IN THE MATTER OF)
)
GPU NUCLEAR CORPORATION)

DOCKET NO. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 123, Rev. 1 for the Oyster Creek Nuclear Generating Station Technical Specifications, filed with the United States Nuclear Regulatory Commission on February 11, 1985, has this day of February 11, 1985, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Henry A. Delo, Jr.
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

By 
Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dsm
#0661A

Nuclear

GPU Nuclear Corporation

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Writer's Direct Dial Number:

February 11, 1985

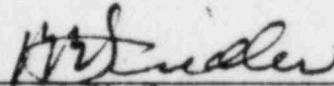
The Honorable Henry A. Delo, Jr.
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

Dear Mayor Delo:

Enclosed herewith is one copy of the Technical Specification Change Request No. 123, Rev. 1 for the Oyster Creek Nuclear Generating Station Operating License.

This document was filed with the United States Nuclear Regulatory Commission on February 11, 1985.

Very truly yours,



Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dam
Enclosure
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OYSTER CREEK NUCLEAR GENERATING STATION
(DOCKET NO. 50-219)
PROVISIONAL OPERATING LICENSE NO. DPR-16

Applicant hereby requests the Commission to reissue Appendix A to the above captioned license:

1) Section to be changed:

Administrative changes indicated below are applicable to Appendix A.

2) Extent of change:

Remove detailed ISI and IST requirements of Section 4.3 and replace with Commission's recommended wording.

3) Changes requested:

As indicated on modified pages 4.3-1, 4.3-2, 4.3-3 and 4.3-4. Delete existing table 4.3-1, renumber existing table 4.3.2, "4.3.1" and appropriately change/remove references to the two tables within the text.

4) Discussion:

This Technical Specification Change Request modifies Section 4.3 by deleting detailed ISI and IST requirements and replacing them with general references to 10 CFR 50.55a and ASME Boiler and Pressure Vessel Code, Section XI. This is in response to a Commission recommendation and will reduce the complexity of administering the programs.

OYSTER CREEK NUCLEAR GENERATING STATION
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 123

Pursuant to 10 CFR 50.91 an analysis concerning significant hazards considerations is provided below:

1) Section to be changed:

Administrative changes indicated below are applicable to Section 4.3 of Appendix A.

2) Extent of change:

Paragraphs 4.3.B and 4.3.C have been changed to reflect the Commission's recommended wording for the conduct of inservice inspection and inservice testing. Paragraph 4.3.D has been revised since the testing requirements for replacement and repaired safety valves are specified in Section XI of the ASME Boiler and Pressure Vessel Code, as stated in Paragraph 4.3.C. The bases have also been changed accordingly. Table 4.3.1 has been removed since it has been superceded by the Oyster Creek Inservice Inspection Program for the second 10-year interval. The existing table 4.3.2 has been renumbered 4.3.1 along with the references to it.

3) Discussion:

Examples of amendments that are considered not likely to involve significant hazards considerations were provided in the Federal Register on April 6, 1983 (48FR 14870). Technical Specification Change Request No. 123 meets the provisions of example (i) (as referenced above) in that it is an administrative change. Copies of modified pages 4.3-1, 4.3-2, 4.3-3, and 4.3-4 are being provided for clarification purposes and the convenience of the Commission. The Commission has recommended the wording employed in modified paragraphs 4.3.B and 4.3.C and the elimination of program details elsewhere in the text. This change will make clearer the inservice testing and inspection requirements as described in 10 CFR 50.55a and Section XI of the ASME Boiler and Pressure Vessel Code, the details of which are implemented by the inservice inspection and testing programs for Oyster Creek. The proposed text will eliminate the need for frequent changes to the Technical Specifications whenever program changes or requests for relief are made.

4) Determination:

We have determined that the subject change request involves no significant hazards in that operation of the Oyster Creek Nuclear Generating Station in accordance with Technical Specification Change Request No. 123, Rev. 1, would not:

- 1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- 2) Create the possibility of a new or different kind of accident from any previously evaluated; or
- 3) Involve a significant reduction in a margin of safety.