

Director, Office of Resource Management
December Monthly Operating Report
January 14, 1985

ATTACHMENT I
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50/395
UNIT V. C. SUMMER I
DATE 01/11/85
COMPLETED BY G. A. Loignon
TELEPHONE (803) 345-5209

MONTH DECEMBER 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-14</u>
2.	<u>-12</u>
3.	<u>-14</u>
4.	<u>-17</u>
5.	<u>-13</u>
6.	<u>-14</u>
7.	<u>-28</u>
8.	<u>-28</u>
9.	<u>-36</u>
10.	<u>-19</u>
11.	<u>-16</u>
12.	<u>-18</u>
13.	<u>-35</u>
14.	<u>-35</u>
15.	<u>-34</u>
16.	<u>-32</u>

17.	<u>-32</u>
18.	<u>-33</u>
19.	<u>-32</u>
20.	<u>40</u>
21.	<u>226</u>
22.	<u>388</u>
23.	<u>510</u>
24.	<u>645</u>
25.	<u>592</u>
26.	<u>658</u>
27.	<u>329</u>
28.	<u>322</u>
29.	<u>644</u>
30.	<u>648</u>
31.	<u>651</u>

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PDR ADOCK 05000395
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ATTACHMENT II
 OPERATING DATA REPORT

DOCKET NO. 50/395
 UNIT V. C. SUMMER I
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OPERATING STATUS

1. Reporting Period: DECEMBER 1984 Gross Hours in Reporting Period: 744
2. Currently Authorized Power Level (MWT): 2775
 Max. Depend. Capacity (MWe-Net): 885
 Design Electrical Rating (MWe-Net): 900
3. Power Level to which restricted (If Any)(MWe-Net): N/A
4. Reasons for Restrictions (If Any): N/A

	<u>THIS MONTH</u>	<u>YR TO DATE</u>	<u>CUMULATIVE</u>
5. Number of Hours Reactor Was Critical	<u>299.9</u>	<u>5,553.4</u>	<u>5,553.4</u>
6. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
7. Hours Generator on Line	<u>270.1</u>	<u>5,365.7</u>	<u>5,365.7</u>
8. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
9. Gross Thermal Energy Generated (MWH)	<u>455,882</u>	<u>13,332,586</u>	<u>13,332,586</u>
10. Gross Electrical Energy Generated (MWH)	<u>145,810</u>	<u>4,432,149</u>	<u>4,432,149</u>
11. Net Electrical Energy Generated (MWH)	<u>124,707</u>	<u>4,192,096</u>	<u>4,192,096</u>
12. Reactor Service Factor	<u>40.3</u>	<u>63.2</u>	<u>63.2</u>
13. Reactor Availability Factor	<u>40.3</u>	<u>63.2</u>	<u>63.2</u>
14. Unit Service Factor	<u>36.3</u>	<u>61.1</u>	<u>61.1</u>
15. Unit Availability Factor	<u>36.3</u>	<u>61.1</u>	<u>61.1</u>
16. Unit Capacity Factor (Using MDC)	<u>18.9</u>	<u>53.9</u>	<u>53.9</u>
17. Unit Capacity Factor (Using Design MWe)	<u>18.6</u>	<u>53.0</u>	<u>53.0</u>
18. Unit Forced Outage Rate	<u>2.2</u>	<u>10.8</u>	<u>10.8</u>

19. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
N/A

20. If Shut Down at End of Report Period, Estimated Date of Startup: N/A

21. Units in Test Status (Prior to Commercial Operation):

	<u>FORECAST</u>	<u>ACHIEVED</u>
Initial Criticality	<u>N/A</u>	<u>10-22-82</u>
Initial Electricity	<u>N/A</u>	<u>11-16-82</u>
Commercial Operation	<u>N/A</u>	<u>01-01-84</u>

ATTACHMENT III
 UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE		DURATION (HOURS)	(1) REASON	(2) METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/ COMMENTS
		F: FORCED	S: SCHEDULED				
11	840928	S		467.5	C	4	11) Refueling, Testing "B" Reactor Coolant Pump Breaker.
12	841220	S		0.3	B	1	12) Turbine Overspeed Test.
13	841227	F		6.1	G	3	13) Reactor Trip Caused By Improper Connection Of Test Instrumentation.

ATTACHMENT IV
NARRATIVE SUMMARY OF OPERATING EXPERIENCE

The Virgil C. Summer Nuclear Station, Unit No. 1, remained shut down for refueling at the beginning of December, 1984.

On December 4, 1984, at 1020 hours, with the Plant in Mode 5, the Residual Heat Removal System Letdown Flow Control Valve failed closed as a result of the loss of its associated 120V AC power source. Residual Heat Removal (RHR) suction relief valves opened at least twice. Pressurizer Power Operated Relief Valves (PORV) were manually opened to maintain the system depressurized. Letdown was restored in approximately twenty minutes.

On December 15, 1984, at 1115 hours, with the Plant in Mode 3, Pressurizer Spray Valve PCV-444D failed open when placed in automatic and would not close in manual control. Reactor coolant pumps "A" and "C" were secured. Air to PCV-444D was secured and the valve was closed. Lowest system pressure was approximately 1985 psig. As pressure increased the rate sensitive PORV PCV-444B opened momentarily at 2245 psig. Alternate spray was used to control pressure and "A" and "C" reactor coolant pumps were restarted.

On December 18, 1984, at 1330 hours, the reactor was taken critical to begin Core Cycle 2 low power physics testing.

On December 19, 1984, at 1330 hours, the Plant was placed in Mode 3 for reactor coolant pump breaker testing. At 1736 hours, a 120V AC inverter failed resulting in loss of its associated 120V AC Instrument Bus. This resulted in an actuation of the Solid State Protection System due to loss of power to the affected Nuclear Instrumentation Channels. Power was returned to the panel at 1750 hours on December 19, 1984.

Subsequent to the Refueling Outage on December 20, 1984, at 1723 hours, the generator was synchronized to the grid.

On December 25, 1984, at 0648 hours, with the unit at 75% power the "A" Condensate Pump tripped. Power was reduced to below 50% and "C" Condensate Pump was used to control the transient.

On December 27, 1984, at 1055 hours and 75% power, an error in installing test equipment caused the feedwater regulating valves to close. A steam generator low level signal coincident with a steam/feed mismatch caused the reactor to trip.

On December 31, 1984, the Plant was operating at 75% power.

OFFSITE DOSE CALCULATION MANUAL

SUMMARY OF CHANGES

REVISION 7

Title, Pages i, ii, and iii: Clerical changes to reflect this revision and increased distribution of the Offsite Dose Calculation Manual (ODCM).

Pages 1.0-13: This change allows greater operational freedom without affecting the manner in which setpoints are determined nor in the way dose is calculated. The ODCM is no less conservative in any of its calculations or in its ability to support such releases. The absolute approbation was previously included because of the potential for large calculated doses if no dilution is available (see equation 31, Page 1.0-28).

Pages 1.0-28, 1.0-34, and 2.0-11: Change corrects typographical errors.