

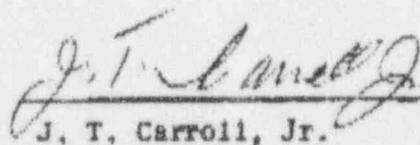
To: James P. O'Reilly  
Directorate of Regulatory Operations  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

From: Jersey Central Power & Light Company  
Oyster Creek Nuclear Generating Station Docket #50-219  
Forked River, New Jersey 08731

Subject: Preliminary Abnormal Occurrence Report No. 73-28

The following is a preliminary report being submitted  
in compliance with the Technical Specifications,  
paragraph 6.6.2.

Preliminary Approval:

  
\_\_\_\_\_  
J. T. Carroll, Jr.                      10/31/73  
Date

cc: Mr. A. Giambusso

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Preliminary  
Abnormal Occurrence  
Report No. 73-28

SUBJECT: Leakage from the reactor building closed cooling water system to the discharge canal via the service water system.

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15C. Notification of this event, as required by the Technical Specifications, paragraph 6.6.2.a, was made to AEC Region I, Directorate of Regulatory Operations, by telephone on Tuesday, October 30, 1973, at 1625, and by telecopier on Wednesday, October 31, 1973, at 1605.

SITUATION: On September 14, 1973, leakage into the reactor building closed cooling water system from a leak in the fuel pool cooling system was identified and isolated. As a result, the closed cooling water system activity was analyzed on several occasions with the highest concentration calculated to be  $1.9 \times 10^{-4}$   $\mu\text{Ci/ml}$  in a sample taken on September 26, 1973. The activity was identified to be primarily  $\text{Cs}^{134}$  and  $\text{Cs}^{137}$ .

Plant personnel subsequently began investigating methods of removing the activity from the closed cooling water system, but these have not yet been implemented.

On October 19, 1973, leakage from the closed cooling water system was noted as indicated by a decreasing level in the reactor building closed cooling water system surge tank of up to 4-4.5"/hr ( $\approx 1.0$  gpm).

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Preliminary  
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October 29, 1973

CAUSE: The cause of the release was leaking tubes in the #1-2 reactor building closed cooling water heat exchanger.

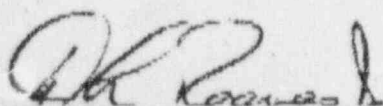
REMEDIAL ACTION:

Operations personnel immediately began isolating various components of the closed cooling water system in order to identify the source of leakage. As noted before, the rate of leakage from the system was conservatively estimated to be 1.0 gpm. On October 22, 1973, a sample of the closed cooling water system indicated a concentration of  $6.3 \times 10^{-5}$   $\mu\text{Ci/ml}$ , indicating that leakage had occurred during the period of September 26, 1973 through October 22, 1973. On October 29, 1973, RBCCW heat exchanger #1-2 was identified as the source of leakage. Prior to this date, there was not assurance that leakage was to the environment. A sample taken on October 30, 1973, following repairs to the heat exchanger, was analyzed to be  $3.5 \times 10^{-5}$   $\mu\text{Ci/ml}$ .

SAFETY SIGNIFICANCE:

A full investigation into the safety significance is now underway. It is apparent, however, that the release rate of liquid effluents into the discharge canal was significantly below the concentrations given in Appendix B, Table II, Column D, of 10CFR20. (10CFR20 limits are  $9 \times 10^{-6}$   $\mu\text{Ci/ml}$  for  $\text{Cs}^{134}$  and  $2 \times 10^{-5}$   $\mu\text{Ci/ml}$  for  $\text{Cs}^{137}$ .)

Prepared by:



Date:

10/31/73