

OYSTER CREEK NUCLEAR GENERATING STATION
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 132

Applicant hereby requests the Commission to change Appendix A to the above captioned license as follows:

1. Sections to be changed:

Section 3.13 Table 3.13.1
Section 4.13 Table 4.13.1

2. Extent of Changes

To incorporate the requirements of accident monitoring instrumentation as required by NUREG 0737 and Generic Letter 83-36 and to clarify the table format of the relief valve position indicators as presented in Table 3.13.1. In addition, the Technical Specification change provides consistency in limiting conditions for operation between all accident monitoring instrumentation.

3. Changes Requested

Add revised page 3.13-1 and add new page 3.13-1a, replace old pages 3.13-3, 4.13-1 and 4.13-2 with new pages (same numbers).

4. Discussion

Technical Specification Change Request No. 132 identifies NUREG-0737 required modifications which were installed during our recent refueling outage. Copies of the modified pages are provided. This change will provide the appropriate action requirements for inoperable containment monitors and surveillance requirements for the above instrumentation.

In addition, this Technical Specification changes (for the purpose of clarification) the format in which the operability requirements of the Relief Valve Position Indicators are presented in Table 3.13.1. This Technical Specification also provides consistency in limiting conditions for operation between all accident monitoring instrumentation.

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OYSTER CREEK NUCLEAR GENERATING STATION
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 132

Pursuant to 10CFR 50.91 an analysis concerning significant hazards considerations is provided below:

1. Sections to be changed:

3.13, 4.13, Table 3.13.1, and Table 4.13.1

2. Extent of changes:

To incorporate the requirements of accident monitoring instrumentation as required by NUREG 0737 and Generic Letter 83-36 and to clarify the table format of the relief valve position indicators as presented in Table 3.13.1. In addition, the Technical Specification change provides consistency in limiting conditions for operation between all accident monitoring instrumentation.

3. Discussion:

To incorporate the requirements of NUREG 0737 as requested by Generic Letter 83-36 which identifies those items for which Technical Specifications are required, and to provide consistency and clarification between all accident monitoring instrumentation. Examples of amendments that are considered not likely to involve significant hazards considerations were provided in the Federal Register on April 6, 1983 (48FR 14870). Technical Specification Change Request No. 132 meets the provisions of example (ii) (as referenced above) in that this change constitutes an additional limitation, restriction, or control not presently included in the Technical Specifications. Copies of modified pages are provided. These changes will provide the appropriate limiting conditions for operation and surveillance requirements in the Technical Specifications for additional accident monitoring instrumentation.

4. Determination:

We have determined that the subject change request involves no significant hazards in that operation of the Oyster Creek Nuclear Generating Station in accordance with Technical Specification Change Request No. 132 would not:

1. Involve a significant increase in the probability or the consequence of an accident previously evaluated; or
2. Create the possibility of a new or different kind of accident from any previously evaluated; or
3. Involve a significant reduction in a margin of safety.

3.13 ACCIDENT MONITORING INSTRUMENTATION

Applicability: Applies to the operating status of accident monitoring instrumentation.

Objective: To assure operability of accident monitoring instrumentation.

Specification: A. Relief Valve Position Indicators

1. The accident monitoring instrumentation channels shown in Table 3.13.1 should be operable when the mode switch is in the Startup or Run positions.
2. With the number of operable accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.13.1, either restore the inoperable channels to operable status within 7 days, or place the reactor in the shutdown condition within the next 24 hours.
3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels Operable requirements of Table 3.13.1, either restore the inoperable channel(s) to the operable status within 48 hours, or place the reactor in the shutdown condition within the next 24 hours.

B. Safety Valve Position Indicators

1. During power operation, both primary* and backup** safety valve monitoring instruments are required to be operable except as provided in 3.13.B.2 and 3.13.B.3.
 2. If either the primary* or backup** accident monitoring instruments on a valve become inoperable, the primary* accident monitoring instrument on an adjacent valve must be operable, and its set point appropriately reduced.
 3. If both the primary* and backup** accident monitoring instruments on a valve become inoperable and the primary* accident monitoring instrument on an adjacent valve is operable, either restore the inoperable channel(s) to an operable status within 7 days, or place the reactor in the shutdown condition within the next 24 hours.
 4. If the requirements of Section 3.13.B.2 or 3.13.B.3 cannot be met within 48 hours, place the reactor in the shutdown condition within the next 24 hours.
- C. In the event that any of these monitoring channels become inoperable, they shall be made operable prior to startup following the next cold shutdown.

* Acoustic Monitor

** Thermocouple

D. Wide Range Torus Water Level Monitor

1. Two wide range torus water level monitor channels should be continuously indicated in the control room during Power Operation.
2. With the number of operable accident monitoring channels less than the total Number of Channels shown in Table 3.13.1, restore the inoperable channel(s) to Operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.
3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels operable requirements of Table 3.13.1 restore the inoperable channel(s) to operable status within 48 hours or place the reactor in the shutdown condition within the next 24 hours.

E. Wide Range Drywell Pressure Monitor

1. Two Wide Range Drywell Pressure monitor channels should be continuously indicated in the control room during Power Operation.
2. With the number of operable accident monitoring channels less than the total Number of Channels shown in Table 3.13.1, restore the inoperable channel(s) to Operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.
3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels operable requirements of 3.13.1 restore the inoperable channel(s) to operable status within 48 hours or place the reactor in the shutdown condition within the next 24 hours.

F. Drywell H₂ Monitor

1. Two drywell hydrogen monitor channels should be capable of continuously indicating in the control room during power operation.
2. With the number of operable channels less than the total number of channels shown in Table 3.13.1, restore the inoperable channel to operable status within 30 days or place the reactor in the shutdown condition within the next 24 hours.
3. With the number of operable channels less than the minimum channels operable requirements of Table 3.13.1, restore at least one channel to operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.

Bases:

The operability of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with NUREGs 0578 and 0737.

TABLE 3.13.1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Relief Valve Position Indicator (See Note 1)	2/valve	1/valve
2. Wide Range Drywell Pressure Monitor (PT/PR-53 & 54)	2	1
3. Wide Range Torus Water Level Monitor (LT/LR-37 & 38)	2	1
4. Drywell H ₂ Monitor	2	1

Note 1) Position indicator consists of one acoustic monitor and one thermocouple per valve.

4.13 ACCIDENT MONITORING INSTRUMENTATION

- Applicability: Applies to surveillance requirements for the accident monitoring instrumentation
- Objective: To verify the operability of the accident monitoring instrumentation.
- Specification:
- A. Safety & Relief Valve Position Indicators
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.
 - B. Wide Range Drywell Pressure Monitor
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.
 - C. Wide Range Torus Water Level Monitor
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.
 - D. Drywell H₂ Monitor
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.

Bases:

The operability of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with NUREGs 0578 and 0737.

TABLE 4.13.1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Relief and Safety Valve Position Indicator (Primary Detector*)	A	B
Relief and Safety Valve Position Indicator (Backup Indications**)	A	B
2. Wide Range Drywell Pressure Monitor (PT/PR 53 & 54)	A	C
3. Wide Range Torus Water Level Monitor (LT/LR 37 & 38)	A	C
4. Drywell H ₂ Monitor	A ¹	D

Legend:

A = at least once per 31 days; B = at least once per 18 months (550 days).

C = at least once per 6 months.

D = at least once per 12 months.

1 = Span and Zero using calibration gasses

*Acoustic Monitor

**Thermocouple