

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) R.E. Ginna Nuclear Power Plant, Unit No. 1 DOCKET NUMBER (2) 05000244 PAGE (3) 1 OF 02

TITLE (4) Loss of Control Rod Position Indication System

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)		
09	28	84	84	011	00	10	28	84		05000		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9)	20.402(b)	20.406(e)	80.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 100	20.406(a)(1)(i)	80.36(e)(1)	80.73(a)(2)(v)	73.71(e)
	20.406(a)(1)(ii)	80.36(e)(2)	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 385A)
	20.406(a)(1)(iii)	X 80.73(a)(2)(i)	80.73(a)(2)(vii)(A)	
	20.406(a)(1)(iv)	80.73(a)(2)(ii)	80.73(a)(2)(vii)(B)	
	20.406(a)(1)(v)	80.73(a)(2)(iii)	80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12) NAME: E.C. Edgar, I&C Supervisor TELEPHONE NUMBER: 315 524-4446

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	IU	-CORNPI	2917	N					

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) X NO

EXPECTED SUBMISSION DATE (15) MONTH: 06 DAY: 01 YEAR: 85

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 28, 1984 and October 11, 1984, with the plant operating at 100% power, the Control Rod Position Indication System was made inoperable for the corrective maintenance of the + 13 VDC power circuit. Each time the system was inoperable for less than one hour.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8   4	-   0   1   1	-   0   0	0   2	OF 0   2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 28, 1984 the Control Rod Position Indication System (RPI's) was made inoperable for 58 minutes, 10 minutes with loss of indication and 48 minutes for RPI alignment, to perform corrective maintenance of an unstable + 13 VDC power source. The DC voltage is used by the operational amplifiers that condition rod position signals for the RPI's. As the DC voltage level changed so would the RPI indications. The indication change was within Technical Specification limits. The + 13 VDC power supply, a plug-in device, was replaced with a substitute supply whose specifications exceeded those of the original supply. A substitute was used because a direct replacement was not available. The substitute was wired directly to the terminal deck, where the DC voltage is daisy chained, bypassing the plug-in socket. All RPI's and voltage levels were stable after energization.

The original supply was tested at full load and continuously monitored for three days with no change in output even when ambient temperature was elevated intentionally. Based on the satisfactory bench test results it has been concluded that the original supply was not the problem, and the most likely cause for the unstable DC voltage is a faulty power supply socket or associated wiring from the socket to the terminal deck.

On October 11, 1984 the RPI's were made inoperable again for 18 minutes in an attempt to identify and correct the problem. Nothing was immediately evident so the system was returned to operable status. ie: with the temporary power supply still in service.

Based on the importance of the RPI's to the operators and to the safe operation of the plant, and considering the substitute supply specifications exceeds those of the original supply all further corrective actions will be delayed until the next plant outage.



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TELEPHONE  
AREA CODE 716 546-2700

October 28, 1984

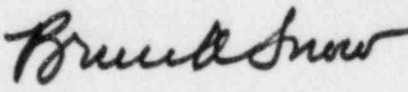
U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: LER 84-011, Inoperable Rod Position Indicating System  
R.E. Ginna Nuclear Power Plant, Unit No. 1  
(Docket No. 50-244)

Document Control Desk:

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (i) (B) "Any operation or condition prohibited by the plant's Technical Specifications" the attached Licensee Event Report LER-84-244 is hereby submitted.

Very Truly Yours,

*for*   
Roger W. Kober

xc: Dr. Thomas E. Murley

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