



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated July 25, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:


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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 11, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise the Appendix "A" Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
66	66
67	67
68	68
69	69
119	119

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TABLE 3.2-2

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design For Both Trip Systems	Remarks
1	2	Reactor Low-Low Water Level	≥ -38 in. indicated level (≥ 126.5 in. above the top of active fuel)	4 HPCI & RCIC Inst. Channels	Initiates HPCI, RCIC & SGTS.
2	2	Reactor Low-Low-Low Water Level	≥ -146.5 in. indicated level (≥ 18 in. above the top of active fuel)	4 Core Spray & RHR Instrument Channels	Initiates Core Spray, LPCI, and Emergency Diesel Generators.
				4 ADS Instrument Channels	Initiates ADS in conjunction with confirmatory low level, 120 second time delay and LPCI or Core Spray pump discharge pressure interlock if not inhibited by ADS override switches.
3	2	Reactor High Water Level	$\leq +58$ in. indicated level (≤ 222.5 in. above the top of active fuel)	2 Inst. Channels	Trips HPCI and RCIC Turbines.
4	1	Reactor Low Level (inside shroud)	$> +352$ in. above vessel zero (≥ 0 in. above the top of active fuel)	2 Inst. Channels	Prevents inadvertent operation of containment spray during accident condition.

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TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design for Both Trip Systems	Remarks
5	2	Containment High Pressure	$1 < p < 2.7$ psig	4 Inst. Channels	Prevents inadvertent operation of containment spray during accident condition.
6	1	Confirmatory Low Level	≥ 12.5 in. indicated level (≥ 177 in. above the top of active fuel)	2 Inst. Channels	ADS Permissive in conjunction with Reactor Low-Low-Low Water Level.
7	2	High Drywell Pressure	≤ 2.7 psig	HPCI Inst. Channels	Initiates Core Spray LPCI, HPCI & SGTS.
8	2	Reactor Low Pressure	≥ 450 psig	4 Inst. Channels	Permissive for opening Core Spray and LPCI Admission valves.

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TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instru- ment Channels Pro- vided by Design for Both Trip Systems	Remarks
9	1	Reactor Low Pres- sure	$50 \leq p \leq 75$ psig	2 Inst. Channels	In conjunction with PCIS signal permits closure of RHR (LPCI) injection valves.
10					
11	THIS ITEM INTENTIONALLY BLANK				
12	1 (See Note 3)	Core Spray Pump Start Timer (each loop)	11 ± 0.6 sec.	1 Inst. Channel	Initiates starting of core spray pumps. (each loop)

TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design for Both Trip Systems	Remarks
13	1 (See Note 3)	RHR Pump Start Timer 1st Pump (A Loop) 1st Pump (B Loop) 2nd Pump (A Loop) 2nd Pump (B Loop)	$1.0 + 0.5 (-) 0$ sec. $1.0 + 0.5 (-) 0$ sec. 6.0 ± 0.5 sec. 6.0 ± 0.5 sec.	1 Inst. Channel 1 Inst. Channel 1 Inst. Channel 1 Inst. Channel	Starts 1st Pump (A Lo Starts 1st Pump (B Lo Starts 2nd Pump (A Lo Starts 2nd Pump (B Lo
14	1	Auto Blowdown Timer	$120 \text{ sec} \pm 5 \text{ sec.}$	2 Inst. Channels	Initiates ADS, in conjunction with Low-Low Reactor Water Level, and LPCI or Core Spray Pump discharge pressure interlock, if not inhibited by ADS override switches.
15	2	RHR (LPCI) Pump Discharge Pressure Interlock	$125 \text{ psig} \pm 20 \text{ psig}$	4 Inst. Channels	Defers ADS actuation pending confirmation of low pressure core cooling system operation.

D. Automatic Depressurization System
(ADS)

1. The ADS shall be operable whenever the reactor pressure is greater than 100 psig, and irradiated fuel is in the reactor vessel and prior to reactor startup from a cold condition, except as specified below:
 - a. From and after the date that one of the seven relief/safety valves of the ADS is made or found to be inoperable for any reason while it is required, continued reactor operation is permissible only during the succeeding 30 days unless repairs are made and provided that during such time the HPCI System is operable.
 - b. From the time that more than one of the seven relief/safety valves of the ADS are made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding 24 hrs. unless repairs are made and provided, that

D. Automatic Depressurization System
(ADS)

1. Surveillance of the Automatic Depressurization System shall be performed during each operating cycle as follows:
 - a. A simulated automatic initiation which opens all pilot valves.
 - b. Manually open each relief/safety valve while bypassing steam to the condenser and observe a $\geq 10\%$ closure of the turbine bypass valves, to verify that the relief/safety valve has opened.
 - c. A simulated automatic initiation which is inhibited by the override switches.