OPERATING DATA REPORT

DOCKET NO. 50-334
DATE 1/7/85
COMPLETED BY P. Smith
TELEPHONE 412-643-1825

OPERATING STATUS

1. Unit Name: Beaver Valley Power Station, Unit #1
2. Reporting Period: December 1984
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 923
5. Design Electrical Rating (Net MWe): 835
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	8784	76,008
12. Number Of Hours Reactor Was Critical	0	6476.3	37,355.7
13. Reactor Reserve Shutdown Hours	0	0	4,482.8
14. Hours Generator On-Line	0	6304.1	36,082.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	15,808,975	83,398,514
17. Gross Electrical Energy Generated (MWH)	0	5,065,500	26,494,400
18. Net Electrical Energy Generated (MWH)	-7870	4,735,955	24,624,843
19. Unit Service Factor	0	71.8	49.7
20. Unit Availability Factor	0	71.8	49.7
21. Unit Capacity Factor (Using MDC Net)	0	66.6	43.4
22. Unit Capacity Factor (Using DER Net)	0	64.6	42.1
23. Unit Forced Outage Rate	0	3.0	27.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Plant shutdown for 4th refueling began October 11 at 1500 hours.
Scheduled duration: 86 days

25. If Shut Down At End Of Report Period, Estimated Date of Startup: January 4, 1985
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

IE24
1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-334
UNIT BVPS Unit #1
DATE 1/7/85
COMPLETED BY P. Smith
TELEPHONE (412) 643-1825

MONTH December 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-334
UNIT NAME BVPS Unit #1
DATE 1/7/85
COMPLETED BY P. Smith
TELEPHONE (412) 643-1825

REPORT MONTH December 1984

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	12/1/84	S	744	C	4	N/A	zz	zzzzzzz	Station remained shutdown for 4th Refueling Outage.

[1] F: Forced
S: Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continued From Previous Month
5-Reduction
9-Other

[4] Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

[5] Exhibit I - Same Source

(9/77)

FOURTH REFUELING STEAM GENERATOR INSPECTION

During the Fourth Refueling Outage, the primary side of the tubing in 'A' Steam Generator was examined by Westinghouse Electric Corporation. The M1Z-18, multi-frequency eddy current system, was utilized for the inspection of all 3388 tubes in 'A' Steam Generator. The tubes were inspected full length, that is, from the tubesheet on the hot leg side over the u-bend region to the tubesheet on the cold leg side. Twenty-four (24) of the 3388 tubes examined contained through-wall defect indications in excess of the BVPS Technical Specifications limit of 40 percent.

Concurrent with the eddy current examination described above, a visual examination of the secondary-side tube sheet region of all three steam generators was conducted. The visual examination included the assessment of the condition of the secondary side of the tubes in the tube lane, the flow slot, and the peripheral areas. A weld rod (4-inch long) was found in 'A' Steam Generator and a piece of flexitallic gasket was found in 'B' Steam Generator each was subsequently removed. No other tube defects were noted by the contractor, Westinghouse Electric Corporation.

The following twenty-four (24) tubes in 'A' Steam Generator were removed from service by mechanically plugging both ends of the tube. All the tubes removed from service contained a defect in excess of the 40 percent limit.

Tube Location Row	Column	Defect Size %Twd	Defect Location
1	2	48	18 inches up from the hot leg tubesheet
1	3	61	18 inches up from the hot leg tubesheet
27	11	47	2nd Support up on the Cold Leg Side
29	12	42	3rd Support up on the Cold Leg Side
32	17	49	1st Support up on the Cold Leg Side
36	21	42	2nd Support up on the Cold leg side
39	25	45	3rd Support up on the Cold leg side
41	26	49	3rd Support up on the Cold Leg Side
41	28	42	2nd Support up on the Cold Leg Side
43	32	46	3rd Support up on the Cold Leg Side
44	34	42	3rd Support up on the Cold Leg Side
43	36	48	3rd Support up on the Cold Leg Side
44	36	45	2nd Support up on the Cold Leg Side
43	37	43	1st Support up on the Cold Leg Side
44	39	42	2nd Support up on the Cold Leg Side

Tube Row	Location Column	Defect Size %Twd	Defect Location
45	43	43	2nd Support up on the Cold Leg Side
26	44	40	4th Support up on the Cold Leg Side
33	44	63	4th Anti-vibration bar
33	48	60	4th Anti-vibration bar
24	62	45	3rd Anti-vibration bar
1	90	69	18 inches up from hot leg tubesheet
1	91	87	18 inches up from hot leg tubesheet
1	92	77	18 inches up from hot leg tubesheet
1	93	41	18 up inches from hot leg tubesheet

Upon completion of all plugging and retrieval activities, the Steam Generators were returned to service.

Duquesne Light

Nuclear Division
P.O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

January 7, 1985

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
Monthly Operating Report

United States Nuclear Regulatory Commission
Director, Office of Management Information & Program Control
Washington, D.C. 20555

Gentlemen:

In accordance with Appendix A, Technical Specifications, the Monthly Operating Report is submitted for the month of December 1984.

Also, please find attached the results of the Fourth Refueling Steam Generator Tube Inspection in accordance with Surveillance Requirement 4.4.5.5, Technical Specifications.

Very truly yours,

J. J. Carey/WDS
Vice President
Nuclear Group

Enclosures

cc: NRC Regional Office, King of Prussia, PA

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Narrative Summary of Monthly
Operating Experience - December 1984

December 1 through December 10	The station was in cold shutdown for 4th Refueling and major modification.
December 11 through December 15	At 0920 hours on the 11th RCS heatup to approximately 104°F was begun, while pressure was maintained at ambient conditions.
December 16 through December 21	On the 16th a steam bubble was established in the pressurizer at 1115 hours. RCS temperature was 140°F pressure was 150 psig. Filling of the Feedwater Heaters and the Condenser began at 2045 hours on the 16th. At 2200 hours on the 19th began RCS pressurization to 250 psig. Continued RCS pressurization reaching 300 psig on the 21st.
December 22	At 1719 hours began pulling containment vacuum. Stopped pulling containment vacuum at 1850 hours with pressure at 12.3 psia due to a required containment entry to check cable connections for rod B-8.
December 23	At 0930 hours reinitiated drawing containment vacuum until 1230 hours when operations were secured with pressure at 9.9 psia. At 1415 hours a spurious reactor trip occurred while withdrawing shutdown banks. Bank "A" was at 225 steps while Bank "B" was at 5 steps. The cause was an over-temperature ΔT spike that occurred during reinstallation of low level temeprature loop amplifiers. A maintenance surveillance procedure on Power Range Detector, N-41 was in progress at the time with loop I bistable tripped. At 1759 hours began RCS heatup to approximately 250°F. RCS pressure was 300 psig. Started Reactor Coolant Pump 1A at 1810 hours. Entered Mode 4 at 1852 hours.
December 24 through December 27	Began plant cooldown at Mode 5 and repressurization of containment. On the 27th entered Mode 5 at 0634 hours. Started drawing containment vacuum at 0915 hours. Began plant heatup to Mode 4 at 1307 hours and entered Mode 4 at 1426 hours. Continued with heatup toward Mode 3.
December 28	Started 1B Reactor Coolant Pump at 0115 hours. Entered Mode 3 at 0330.
December 29 through December 31	The station was in Operational Mode 3, hot standby. The RCS temeprature and pressure were being increased to normal operating level. Surveillance testing was in progress.

Major Maintenance - December 1984

1. 10-year Inservice Inspection of welds in Containment
2. Retube of the Main Condenser
3. Inspection of Main Unit Generator
4. Replacement of First Point Feedwater Heaters [FW-E-1A, 1B]
5. Overhaul of Main Feedwater Pump [FW-P-1A]
6. Reactor Coolant Pump [RC-P-1B] Seal Inspection and Maintenance