

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 2	DOCKET NUMBER (2) 050000260	PAGE (3) 1 OF 02
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TITLE (4)
Main Steam Isolation Valve Excessive Seat Leakage

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
09	22	84	84	007	00	10	19	84			050000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 000	20.406(a)(1)(i)	50.36(a)(1)	50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.36(a)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 306A) Part 21
	20.406(a)(1)(iii)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	X 50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Jimmy B. Walker	TELEPHONE NUMBER 210571291-3865
AREA CODE	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	S	B	I S V A	585	Y				

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During performance of Refueling Surveillance Testing, it was determined that the "B" and "C" main steam line isolation valves had excessive leakage above Technical Specification limits.

The valve poppet will be machined and the valve seat lapped to ensure proper seating and acceptable leakage prior to unit startup.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		0 0 7	0 0	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 385A's) (17)

Unit 1 was operating at 99.6 percent power and unit 2 and 3 were in a refueling outage. This event affected only unit 2.

During the performance of Surveillance Instruction (Primary Containment Isolation Valve Leak Rate Test), it was determined that the isolation valve (ISV) on the B and C main steam (SB) lines were found to have excessive leakage (greater than 11.5 SCFH).

The A and D line isolation valves were tested and only required valve packing adjustments to bring them into Technical Specification limits.

The valve poppets on B and C main steam lines will be machined and the valve seat lapped to ensure proper seating. Additional valve poppet guide's buildup were added to the A and D line inboard valves due to their physical arrangement with the horizontal plane. The added metal to the lower poppet guides has reduced the amount of valve seating surface erosion providing for better leak rate results. An engineering change notice has been approved to add additional buildup metal to the bottom three inches of the lower guides in all main steam isolation valves.

This event is reportable under 10 CFR 50, Part 21. Individual valve leakages will be submitted in the local leak rate test report.

Responsible Plant Section - N/A

Previous Similar Events -

- BFRO 50-259/77023, 78034, 80003, 81014, 83020
- BFRO 50-260/79077, 80092, 82030, 82037
- BFRO 50-296/78025, 79014, 80058, 80059, 81073, 83054

TENNESSEE VALLEY AUTHORITY
Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

October 19, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-09 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/84007

The enclosed report provides details that concern the main steam isolation valve excessive seat leakage during refueling surveillance testing. This event is Part 21 reportable. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(ii).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Resident Inspector, BFN

This was prepared principally by Jimmy B. Walker.

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