AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date Feb. 10,1985
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month January 1985

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	7	17	1105
2	838	18	1080
3	1088	19	1096
4	1100	20	1102
5	1086	21	1093
6	1047	22	1087
7	1101	23	1110
8	1104	24	1112
9	1075	25	1110
10	1052	26	1098
11	1097	27	1094
12	1110	28	1110
13	1092	29	1108
14	1102	30	1110
15	1103	31	1104
16	1099		

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OPERATING DATA REPORT

Docket No. 50-272
Date Feb. 10, 1985
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	January 1985	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1170*	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason * Previous value reported was the turbine rating, not the generator rating as required.

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	744	66553
12. No. of Hrs. Reactor was Critical	726.6	726.6	36559.1
13. Reactor Reserve Shutdown Hrs.	0	0	3088.4
14. Hours Generator On-Line	723.7	723.7	34882.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2382022	2382022	105150019
17. Gross Elec. Energy Generated (MWH)	815390	815390	34731440
18. Net Elec. Energy Generated (MWH)	782864	782864	32880846
19. Unit Service Factor	97.3	97.3	52.4
20. Unit Availability Factor	97.3	97.3	52.4
21. Unit Capacity Factor (using MDC Net)	97.5	97.5	45.8
22. Unit Capacity Factor (using DER Net)	96.5	96.5	45.3
23. Unit Forced Outage Rate	2.7	2.7	32.8

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH January 1985

Completed by J.P. Ronafalvy

Docket No. 50-272
Unit Name Salem No.1
Date Feb. 10,1985
Telephone 609-935-6000
Extension 4455

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
85-010	1-01	F	20.3	A	3	-	HH	VALVEX	Feedwater Regulating Boiler Level Control Valve

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH January 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: February 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1417	Over/Under Voltage Check	Remove over/under voltage check restrictions on breaker 11X, 21X, and 31X closing/reclosing control.
1EC-1747	Hydrogen & Seal Oil	Add liquid level sensor with alarm to indicate high water level in moisture separator located in discharge from loop seal drain vapor extractor.
1ET-1940	4KV Power System	Test the 4KV U.V. transfer scheme on the 1B Vital Bus.
1EC-1990	Auxiliary Feedwater	Change relay setting for #11 and #12 Aux. Feed Pump motors.
1SC-0419	Service Water	Revise motor service water cooling piping to the service water pump motor coolers which will allow quick removal and restoration of piping hangers and associated piping during motor change out.
1SC-1341	Service Water Structure	Replace the ladders located between the Service Water Pump inlets and rotating screens. New ladders should be made of non-corrosive material.

MAJOR PLANT MODIFICATIONS
REPORT MONTH JANUARY 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: February 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1417	The temporary jumpers across the overvoltage relay contacts on the 500 KV line do not affect the performance of any safety related equipment. No unreviewed safety or environmental questions are involved.
1EC-1747	The addition of a level detector on the moisture separator located in the discharge from the loop seal drain vapor extrator of the Hydrogen Seal Oil System will not affect the function or operation of the system or any safety related equipment. No unreviewed safety or environmental questions are involved.
1ET-1940	This DCR is only a test of the 4160 Vital Bus U. V. transfer scheme in order to obtain necessary data for the protective relay evaluation. No unreviewed safety or environmental questions are involved.
1EC-1990	This DCR only involves a relay setpoint change. No unreviewed safety or environmental questions are involved.
1SC-0419	No basic arrangement, function or operating component has been changed per this DCR. Only the cooling water hard piping to the pump motor coolers has been modified to allow for easier change-out of the pump motors. The flex hose used is supplied in accordance with quality assurance requirements of 10CFR50, including seismic testing. No unreviewed safety or environmental questions are involved.
1SC-1341	This DCR replaces carbon steel ladders with stainless steel ladders. Failure of the anchorage units or the ladder assemblies will not affect any system associated with the safe shutdown of either reactor. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
85-01-12-104-3	SMD	1	NO. 11 BORIC ACID TRANSFER PUMP FAILURE DESCRIPTION: BAD SEAL LEAK CORRECTIVE ACTION: REPLACED SEAL
0099120712	SMD	1	1CV346 (CHARGING LINE DRAIN VALVE) FAILURE DESCRIPTION: VALVE IS LEAKING CORRECTIVE ACTION: CUT DRAIN SIDE OF VALVE AND INSTALLED A PIPE CAP OVER THE LINES (PER DR #MD84-3352)
84-11-09-054-6	SMD	1	VALVE 1CV144 FAILURE DESCRIPTION: VALVE LEAKS CORRECTIVE ACTION: INSTALLED NEW BONNET ASSEMBLY WITH HANDWHEEL, STEM AND GASKET
009911930-7		1	VITAL HEAT TRACE HEATER #208 FAILURE DESCRIPTION: HEATER NOT MAINTAINING TEMPERATURE CORRECTIVE ACTION: REPLACED HEAT TAPE; INSULATED
85-01-059-2	SMD	1	16 SW PUMP STRAINER FAILURE DESCRIPTION: SHEAR PIN BROKEN CORRECTIVE ACTION: REPLACED SHEAR KEY AND FILTER ELEMENTS

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
0099104377	SMD	1	NO. 12 B. U. GROUP - PRESSURIZER HEATERS
			FAILURE DESCRIPTION: SEVEN OF THE FOURTEEN ACB'S WILL NOT RESET OR THEY TRIP OUT FREQUENTLY
			CORRECTIVE ACTION: REPLACED TWO OF THE ITE CIRCUIT BREAKERS AND RESET THE OTHERS
0099104385	SMD	1	NO. 11 BAT PUMP
			FAILURE DESCRIPTION: LEAKING WATER EXCESSIVELY THROUGH SEALS
			CORRECTIVE ACTION: REPLACED PUMP
0099125307	SMD	1	VALVE 1VC5
			FAILURE DESCRIPTION: VALVE WON'T CLOSE FROM CONTROL ROOM BEZEL
			CORRECTIVE ACTION: REPLACED SOLENOID VALVE
0099124319	SIC	1	R41A
			FAILURE DESCRIPTION: THE HI/LO FLOW ALARMS UP
			CORRECTIVE ACTION: REPLACED THE APD PUMP
0099123746	SIC	1	1R41C
			FAILURE DESCRIPTION: CHANNEL PEGGING HIGH
			CORRECTIVE ACTION: INSTALLED NEW PINS IN "J" CONNECTORS FOR THE CONTROLLER AND SCALER

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-09-12-015-1	SIC	1	MAIN STEAM LINE MONITOR
			FAILURE DESCRIPTION: CHANNEL SPIKING
			CORRECTIVE ACTION: REPAIRED DRAWER AND DRIED OUT THE DETECTOR CONNECTORS, AND CABLES
84-12-30-015-7	SMD	1	104 PANEL
			FAILURE DESCRIPTION: POWER LOSS TO THE ALARMS
			CORRECTIVE ACTION: AUDIBLE HORN REPLACED
84-12-24-009-0	SMD	1	INCORE SYSTEM
			FAILURE DESCRIPTION: DETECTORS A AND F ARE DRIFTING EXCESSIVELY
			CORRECTIVE ACTION: REPLACED A AND F CABLES
0099118777	SMD	1	1RC30
			FAILURE DESCRIPTION: BORIC ACID PACKING LEAK
			CORRECTIVE ACTION: VALVE WAS FURMANITED

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
JANUARY 1985

SALEM NO. 1

The period began with the Unit power increasing to full power. Monitoring of the Electro-Hydraulic Control (EHC) System was continuing. On 12/31/84 at 2123 hours, the Unit tripped due to No. 11 Steam Generator Steam Flow/Feed Flow mismatch combined with Steam Generator Low Level. This mismatch was a result of partial closure of No. 11 Steam Generator Feedwater Control valve 11BF19 caused by a malfunctioning solenoid. Following replacement of the solenoid, the Unit was returned to service on 1/1/85 at 2017 hours. Full power operation was achieved on 1/2/85 and continued for the remainder of the period. On 1/11/85, the additional monitoring of the Electro-Hydraulic Control (EHC) System, instituted after the reactor trips of 11/06/84 and 11/11/84, was discontinued. No abnormal readings were identified during the entire monitoring period that could have caused the two (2) reactor trips associated with the EHC System. It has been assumed that the cards replaced, after the trips, contributed to the cause of the reactor trips. No significant freeze problems have occurred during the current extreme cold weather conditions.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: February 10, 1985
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month January 1985

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE 10/1/84

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSEG

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

February 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of January 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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