



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83  
License No. NPF-58

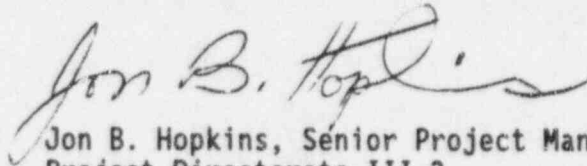
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by The Cleveland Electric Illuminating Company (CEICO), Centerior Service Company, Duquesne Light Company, Ohio Edison Company, OES Nuclear, Inc., Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated February 17, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 83 are hereby incorporated into this license. The Cleveland Electric Illuminating Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jon B. Hopkins, Senior Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of issuance: March 18, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications including the issued but not yet implemented Improved Technical Specifications (ITS) with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-3

3/4 6-4

ITS 3.6-18

Insert

3/4 6-3

3/4 6-4

ITS 3.6-18

## CONTAINMENT SYSTEMS

### PRIMARY CONTAINMENT LEAKAGE

#### LIMITING CONDITION FOR OPERATION

3.6.1.2 Primary containment leakage rates shall be limited to:

- a. An overall integrated leakage rate, except for the main steam line isolation valves#, of less than or equal to  $0.75 L_a$ , where  $L_a$  is 0.20 percent by weight of the primary containment air per 24 hours at  $P_a$ .
- b. A combined leakage rate of less than or equal to  $0.60 L_a$  for all penetrations and all valves, except for main steam line isolation valves# and valves which are hydrostatically leak tested, subject to Type B and C tests when pressurized to  $P_a$ .
- c. Less than or equal to 25 scf per hour\* for any one main steam line through the isolation valves when tested at  $P_a$ .
- d. A combined leakage rate of less than or equal to  $0.0504 L_a$  for all penetrations that are secondary containment bypass leakage paths when pressurized to the required test pressure.
- e. A combined leakage rate of less than or equal to 1 gpm times the total number of containment isolation valves in hydrostatically tested lines which penetrate the primary containment, when tested at greater than or equal to  $1.10 P_a$ .

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, AND 3, with the reactor coolant system temperature greater than 200 °F.

#### ACTION:

- With:
- a. The measured overall integrated primary containment leakage rate, except for the main steam line isolation valves#, exceeding  $0.75 L_a$ , or
  - b. The measured combined leakage rate for all penetrations and all valves except for main steam line isolation valves# and valves which are hydrostatically leak tested, subject to Type B and C tests exceeding  $0.60 L_a$ , or
  - c. The measured leakage rate exceeding 25 scf per hour\* for any one main steam line through the isolation valves, or

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# Exemption to Appendix J of 10 CFR 50.

\* One main steam line may be less than or equal to 35 scf per hour as long as total leakage rate through all four main steam lines is less than or equal to 100 scf per hour. This will remain in effect from the effective date of this amendment until the completion of Operating Cycle 6.

## CONTAINMENT SYSTEMS

### PRIMARY CONTAINMENT LEAKAGE

#### LIMITING CONDITION FOR OPERATION (Continued)

- d. The combined leakage rate for all penetrations that are secondary containment bypass leakage paths exceeding  $0.0504 L_a$ , or
- e. The measured combined leakage rate for all containment isolation valves in hydrostatically tested lines which penetrate the primary containment exceeding 1 gpm times the total number of such valves.

Restore the leakage rate to less than or equal to the above limit(s) within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

#### SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50, except that the provisions of Bechtel Topical Report BN-TOP-1 may be used for Type A tests having a duration less than 24 hours:

- a. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at  $40 \pm 10$  month intervals during shutdown at  $P_a$  during each 10-year service period.
- b. If any periodic Type A test fails to meet  $0.75 L_a$ , the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet  $0.75 L_a$ , a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet  $0.75 L_a$ , at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.6.1.3.9 -----NOTE----- Only required to be met in MODES 1, 2, and 3. -----</p> <p>Verify the combined leakage rate for all secondary containment bypass leakage paths is <math>\leq 0.0504 L_a</math> when pressurized to <math>\geq P_a</math>.</p>	<p>-----NOTE----- SR 3.0.2 is not applicable -----</p> <p>In accordance with 10 CFR 50, Appendix J, as modified by approved exemptions</p>
<p>SR 3.6.1.3.10 -----NOTE----- Only required to be met in MODES 1, 2, and 3. -----</p> <p>Verify leakage rate through each main steam line is <math>\leq 25</math> scfh when tested at <math>\geq P_a</math>. Until the end of Operating Cycle 6, the leakage rate through one main steam line is limited to <math>\leq 35</math> scfh when tested at <math>\geq P_a</math>, as long as the total leakage rate through all four main steam lines is <math>\leq 100</math> scfh.</p>	<p>-----NOTE----- SR 3.0.2 is not applicable -----</p> <p>In accordance with 10 CFR 50, Appendix J, as modified by approved exemptions</p>

(continued)