

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

March 13, 1996
LIC-96-0036

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Reference: Docket No. 50-285

SUBJECT: February 1996 Monthly Operating Report (MOR)

Enclosed please find the February 1996 MOR for Fort Calhoun Station (FCS)
Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,



T. L. Patterson
Division Manager
Nuclear Operations

TLP/d11

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

FEBRUARY 1996
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of February 1996, the Fort Calhoun Station (FCS) operated at a nominal 100% power. Normal plant maintenance, surveillance, equipment rotation activities and scheduled on-line modifications were performed during the month.

Monitoring of the Control Element Drive Mechanism (CEDM) No. 15 mechanical seal leak continued. A planned maintenance outage is scheduled to commence at 2200 hours on March 15, 1996, with a planned duration of seven days. During this mini-outage, the seal will be replaced and other work will be completed to improve station reliability for the upcoming summer.

On February 1st, a supplemental one hour non-emergency notification was made to the NRC pursuant to 10CFR50.72(b)(1)(ii)(B) and (C) to update a reportable event from November 1995 in which FCS was found to be in a condition outside its design basis. Originally, only the motor control center (MCC) 480VAC breakers were considered susceptible to ground induced tripping. Further review concluded that, as a conservative measure, the affected breakers feeding safeguards motors should be included in the group of susceptible breakers. A modification to resolve the 480V breaker issue was finalized in February. Replacement trip devices have been procured with seven of the thirteen most critical breakers scheduled to be replaced during the March mini-outage. Six motor feeder breakers are scheduled to be replaced on-line.

Eight additional incore nuclear detectors failed in February 1996, rendering five of the twenty-eight detector strings inoperable. All failures have occurred in detectors that were installed during the 1995 refueling outage. These failures are under investigation with assistance from ABB/CE and the incore detector vendor.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of February, no power operated relief valve (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The February Reactor Coolant System (RCS) leak rate was relatively steady at approximately 0.2 to 0.3 gpm throughout the month. This leak rate continued the trend established following the reactor trip on August 26, 1995. Repacking all three charging pumps in January and early February due to packing leaks has mitigated short-term increases in the leak rate.

The major contributor of the RCS leakage is "Known" leakage which is collected in both the Reactor Coolant Drain Tank (RCDT) and the Pressurizer Quench Tank (QT). Approximately 0.1 to 0.15 gpm of the total RCS leakage has been classified as "Known" leakage. The primary leakage source for "Known" leakage continues to be CEDM No. 15. In addition to CEDM No. 15, several of the reactor head vent system isolation valves are also leaking through. Head vent leakage is contributing an estimated 0.06 gpm with leakage being collected both in the Containment Sump and the Pressurizer QT. Leakage to the containment sump is considered "Unknown" leakage.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
None	

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF FEBRUARY 1996

- Performed repairs on toxic gas monitors YIT- 6288B and YIT-6286B
- Replaced the ratcheting wrenches on all eight of the Raw Water (RW)/ Component Cooling Water interface valves for the Containment Cooling Units
- Cleaned out the air sparging line for RW Pump AC-10C

6. OPERATING DATA REPORT

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8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	MARCH 07, 1996
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 53306843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: FEBRUARY 1996

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	696.0	1440.0	196634.0
12. Number of Hours Reactor was Critical	696.0	1440.0	155148.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	696.0	1440.0	153420.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1041395.6	2154509.6	204840817.9
17. Gross Elec. Energy Generated (MWH)..	354312.0	733460.0	67667185.2
18. Net Elec. Energy Generated (MWH)....	338525.5	700783.1	64558151.9
19. Unit Service Factor.....	100.0	100.0	78.0
20. Unit Availability Factor.....	100.0	100.0	78.0
21. Unit Capacity Factor (using MDC Net)	101.8	101.8	71.0
22. Unit Capacity Factor (using DER Net);	101.8	101.8	69.4
23. Unit Forced Outage Rate.....	.0	.0	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
A MAINTENANCE OUTAGE IS SCHEDULED TO OCCUR FROM MARCH 15-23, 1996 TO RE-PAIR/REPLACE DEGRADING CEDM MECHANICAL SEALS.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE MARCH 07, 1996
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 53306843

MONTH FEBRUARY 1996

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	486	17	487
2	486	18	486
3	487	19	487
4	487	20	487
5	487	21	487
6	487	22	487
7	487	23	487
8	487	24	486
9	487	25	486
10	486	26	486
11	486	27	486
12	486	28	486
13	486	29	486
14	486	30	N/A
15	486	31	N/A
16	486		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun St.
 DATE March 7, 1996
 COMPLETED BY D. L. Lippy
 TELEPHONE (402) 533-6843

REPORT MONTH February 1996

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
NONE									

- 1
F: Forced
S: Scheduled
- 2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

- 3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

- 4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

- 5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending February 29, 1996

1. Scheduled date for next refueling shutdown. September 21, 1996
2. Scheduled date for restart following refueling. November 2, 1996
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be? Enrichment limit of spent fuel racks is to be increased to at least 4.5 w/o from 4.2 w/o. This is necessary based upon the preliminary Cycle 17 core pattern development.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. N/A
 - c. If no such review has taken place, when is it scheduled? N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. Spent fuel rack enrichment limit change was submitted February 1, 1996.
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. N/A
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 618 Assemblies
 - c) spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 2007 Outage

Prepared by 

Date 3/11/96