10 CFR 50.73

Virginia Electric and Power Company
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117

March 11, 1995

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

NAPS: GSS
Docket No. 50-338
License No. NPF-4

Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 1.

Report No. 50-338/96-001-00

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,

J. A. Stall
Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

R. D. McWhorter
NRC Senior Resident Inspector
North Anna Power Station

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NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
North Anna Power Station Unit 1	05000338	1 OF 4

TITLE (4)
MAIN STEAM AND PRESSURIZER SAFETY VALVE SETPOINTS OUT OF TOLERANCE DUE TO SETPOINT DRIFT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
02	16	96	96	001	00	03	11	96	FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)	6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)	0		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(B)
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(C)
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER
			20.405(a)(1)(iii)	X	50.73(a)(2)(i) (B)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Mr. J. A. Stall	(540) 894-2101

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	AB	RV	D243	Y						
X	SB	RV	C710	Y						

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	YES (If yes, completed EXPECTED SUBMISSION DATE)	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On February 16, 1996, with Unit 1 in Mode 6 (Refueling), the "as found" set pressure for one Main Steam Safety Valve (MSSV) was found to be outside the setpoint tolerance allowed by Technical Specifications (TS). During subsequent testing on February 19 and 20, 1996, the "as found" set pressures for two Pressurizer Safety Valves (PSVs) were also found to be outside the TS setpoint tolerance. These events are reportable pursuant to 10 CFR 50.73 (a) (2) (i) (B) for conditions prohibited by TS.

Five Main Steam Safety Valves from the original test group and three Pressurizer Safety Valves were sent to Wyle Labs for testing to ensure conformance to Technical Specifications requirements. The "as found" set pressures for one MSSV and two PSVs were found to be outside TS tolerances. The Main Steam Safety Valve, upon completion of the additional testing for data gathering and comparison purposes, and the Pressurizer Safety Valves were adjusted and subsequently tested within Technical Specifications limits. Repairs were not required on these valves.

This event posed no significant safety implications because the safety valves would have performed their safety function in the event of an overpressure condition. The health and safety of the public were not affected by this event.

1.0 Description of the Event

Five Main Steam Safety Valves (EIIS System SB, Component RV) from the Original Test Group - A Bank and three Pressurizer Safety Valves (EIIS System AB, Component RV) were sent to Wyle Laboratories for the performance of periodic test procedures 1-PT-70, "Main Steam Code Safety Valve Setpoint Verification" and 1-PT-50, "Pressurizer Code Safety Valve Setpoint Verification" respectively.

On February 16, 1996, with Unit 1 in Mode 6 (Refueling), the "as found" set pressure for Main Steam Safety Valve 01-MS-SV-101A was found to be outside the lift set pressure of 1085 psi +/-3 percent allowed by Technical Specifications 3.7.1.1. Due to the failure of one valve in the test group from A Bank, two additional valves from C Bank were tested with satisfactory results. On February 19 and 20, 1996, the "as found" set pressures for Pressurizer Safety Valves 01-RC-SV-1551C and 01-RC-SV-1551A, respectively, were found to be outside the lift set pressure of 2485 psi +/-1 percent allowed by Technical Specifications 3.4.2 and 3.4.3.1.

2.0 Significant Safety Consequences and Implications

These events posed no significant safety implications. The "as found" set pressure settings for the Safety Valves are bounded by the safety analyses for overpressure transients and the acceptance criteria for overpressure accidents would have been met. The health and safety of the public were not affected at any time during this event.

These events are reportable pursuant to 10 CFR 50.73 (a) (2) (i) (B) for conditions prohibited by Technical Specifications 3.4.2, 3.4.3.1, 3.7.1.1.

3.0 Cause of the Event

The cause of the event has been determined to be setpoint drift. The industry has experienced a history of setpoint drift for safety valves of this type and is not considered to be unusual.

4.0 Immediate Corrective Actions

Upon completion of additional lifts for data gathering and comparison purposes, Main Steam Safety Valve, 01-MS-SV-101A, was adjusted and subsequently tested within the allowable TS setpoint limit tolerances. The results of the additional lifts were within the +/-1 percent tolerance allowed by TS. A visual inspection of the valve internals was performed whereby no abnormalities were identified. Maintenance activities on this valve included lap polishing of the nozzle and disc.

The Pressurizer Safety Valves, 01-RC-SV-1551A and 01-RC-SV-1551C, were adjusted and subsequently tested within the allowable TS setpoint limit tolerances. Maintenance activities were not required on these valves.

Repairs were not required on these Main Steam and Pressurizer Safety Valves.

5.0 Additional Corrective Actions

Deviation Reports were initiated to identify and evaluate the cause of the as-found setpoints being outside TS limits for the Main Steam and Pressurizer Safety Valves.

6.0 Actions to Prevent Recurrence

A Technical Specifications change request has been submitted to the NRC. The proposed Technical Specification changes will increase the tolerance for the Pressurizer Safety Valve as-found lift setting from +/-1 percent to an average of +2/-3 percent with no single valve outside of 3 percent. This will provide the maximum positive tolerance without a reduction in the nominal lift setpoint. All pressurizer safety valves were tested within this proposed criteria.

7.0 Similar Events

Previous similar events where Main Steam Safety Valves have been outside the requirements of Technical Specifications 3.7.1.1 have occurred at North Anna Unit 1 on February 8, 1980 (LER 80-009/03L-0), May 8, 1987 (LER 87-009-01), June 13, 1989 (LER 89-009-00), February 15, 1991 (LER 91-002-00), and January 20, 1993 (LER 93-002-00).

7.0 Similar Events (continued)

Previous similar events where the Pressurizer Safety Valves have been outside the requirements of Technical Specifications 3.4.2 and 3.4.3 have occurred at North Anna Unit 1 on March 2, 1981 (LER 81-040/03L-0), May 6, 1987 (LER 87-008-00), February 5, 1992 (LER 92-002-00), January 20, 1993 (LER 93-002-00), and October 11, 1994 (LER 94-006-00).

8.0 Additional Information

During this period, Unit 2 was operating at 100% power and was not affected by these events.

The following are the Technical Specifications setpoints and "as found" pressures:

Main Steam Safety Valve	Set Pressure (psig)	As Found (psig)
01-MS-SV-101A	1085 +/-32.5	1155

Pressurizer Safety Valves	Set Pressure (psig)	As Found (psig)
01-RC-SV-1551A	2485 +/-24.8	2436
01-RC-SV-1551C	2485 +/-24.8	2539