Virginia Electric and Power Company North Anna Power Station P. O. Box 402 Mineral, Virginia 23117

March 11, 1995

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555 NAPS: GSS Docket No. 50-338 License No. NPF-4

Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 1.

Report No. 50-338/96-001-00

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,

J. A. Stall Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission 101 Marietta Street, N.W. Suite 2900 Atlanta, Georgia 30323

> R. D. McWhorter NRC Senior Resident Inspector North Anna Power Station

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NRC FORM 366	U.S. NUCLE	AR REGULATORY COMMISSION	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95				
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FACILITY	NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)			

		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
North Anna Power Station Unit 1	05000338	96	001	00	2 OF 4
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1.0 Description of the Event

Five Main Steam Safety Valves (EIIS System SB, Component RV) from the Original Test Group - A Bank and three Pressurizer Safety Valves (EIIS System AB, Component RV) were sent to Wyle Laboratories for the performance of periodic test procedures 1-PT-70, "Main Steam Code Safety Valve Setpoint Verification" and 1-PT-50, "Pressurizer Code Safety Valve Setpoint Verification" respectively.

On February 16, 1996, with Unit 1 in Mode 6 (Refueling), the "as found" set pressure for Main Steam Safety Valve 01-MS-SV-101A was found to be outside the lift set pressure of 1085 psi +/-3 percent allowed by Technical Specifications 3.7.1.1. Due to the failure of one valve in the test group from A Bank, two additional valves from C Bank were tested with satisfactory results. On February 19 and 20, 1996, the "as found" set pressures for Pressurizer Safety Valves 01-RC-SV-1551C and 01-RC-SV-1551A, respectively, were found to be outside the lift set pressure of 2485 psi +/-1 percent allowed by Technical Specifications 3.4.2 and 3.4.3.1.

2.0 Significant Safety Consequences and Implications

These events posed no significant safety implications. The "as found" set pressure settings for the Safety Valves are bounded by the safety analyses for overpressure transients and the acceptance criteria for overpressure accidents would have been met. The health and safety of the public were not affected at any time during this event.

These events are reportable pursuant to 10 CFR 50.73 (a) (2) (i) (B) for conditions prohibited by Technical Specifications 3.4.2, 3.4.3.1, 3.7.1.1.

3.0 Cause of the Event

The cause of the event has been determined to be setpoint drift. The industry has experienced a history of setpoint drift for safety valves of this type and is not considered to be unusual.

U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95				
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)		ESTIMATED BURDEN PER RESPONSE TO COMPLY WIT INFORMATION COLLECTION REQUEST. 50.0 HOURS FOI COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORM AND RECORDS MANAGEMENT BRANCH (MNBB 7714). U.S. NU REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, THE PAPERWORK REDUCTION PROJECT (3150-0104), OFF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.					
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4.0 Immediate Corrective Actions

Upon completion of additional lifts for data gathering and comparison purposes, Main Steam Safety Valve, 01-MS-SV-101A, was adjusted and subsequently tested within the allowable TS setpoint limit tolerances. The results of the additional lifts were within the +/-1 percent tolerance allowed by TS. A visual inspection of the valve internals was performed whereby no abnormalities were identified. Maintenance activities on this valve included lap polishing of the nozzle and disc.

The Pressurizer Safety Valves, 01-RC-SV-1551A and 01-RC-SV-1551C, were adjusted and subsequently tested within the allowable TS setpoint limit tolerances. Maintenance activities were not required on these valves.

Repairs were not required on these Main Steam and Pressurizer Safety Valves.

5.0 Additional Corrective Actions

Deviation Reports were initiated to identify and evaluate the cause of the as-found setpoints being outside TS limits for the Main Steam and Pressurizer Safety Valves.

6.0 Actions to Prevent Recurrence

A Technical Specifications change request has been submitted to the NRC. The proposed Technical Specification changes will increase the tolerance for the Pressurizer Safety Valve as-found lift setting from +/-1 percent to an average of +2/-3 percent with no single valve outside of 3 percent. This will provide the maximum positive tolerance without a reduction in the nominal lift setpoint. All pressurizer safety valves were tested within this proposed criteria.

7.0 Similar Events

Previous similar events where Main Steam Safety Valves have been outside the requirements of Technical Specifications 3.7.1.1 have occurred at North Anna Unit 1 on February 8, 1980 (LER 80-009/03L-0), May 8, 1987 (LER 87-009-01), June 13, 1989 (LER 89-009-00), February 15, 1991 (LER 91-002-00), and January 20, 1993 (LER 93-002-00).

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North Anna Power Station Unit (If more space is required, use additional		96	001	00	4 OF 4	
requirements of Technical Unit 1 on March 2, 1981 (L	tinued) here the Pressurizer Safety V Specifications 3.4.2 and 3.4 .ER 81-040/03L-0), May 6, 1 January 20, 1993 (LER 93-0	.3 have 987 (LE	occurred at R 87-008-0	North Anna 0), Februar	a Y	
(LER 94-006-00). <u>8.0</u> Additional Information During this period, Unit 2 we events.	<u>on</u> vas operating at 100% powe	r and w	as not affec	ted by these	е	
The following are the Tech Main Steam Safety Valve 01-MS-SV-101A	nical Specifications setpoint Set Pressure (psig) 1085 +/-32.5	А	as found" pro as Found (p: 155			
Pressurizer Safety Valves 01-RC-SV-1551A 01-RC-SV-1551C	Set Pressure (psig) 2485 +/-24.8 2485 +/-24.8	2	s Found (p: 436 539	sig)		